

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-313

ARKANSAS POWER AND LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 6 to Facility Operating License No. DPR-51, issued to Arkansas Power and Light Company, which revised Technical Specifications for operation of the Arkansas Nuclear One - Unit 1 located in Pope County, Arkansas. The amendment is effective as of its date of issuance.

This amendment (1) modifies the rod withdrawal limit curves to include limitations associated with maintaining potential ejected control rod worth within previously established limits (including following control rod interchange) and limitations associated with maintaining shutdown margin, (2) deletes the separate specification on maximum inserted control rod worths, but includes the limits and bases therefor in (1) above, (3) incorporates an additional restriction on the regulating control rod positions prior to criticality to assure that the ejected rod worth does not exceed 1% delta k/k at hot zero power, and (4) permits the rod limits to be exceeded for a maximum period of four hours, provided that shutdown margin requirements are maintained and corrective measures are taken immediately to achieve a rod pattern consistent with the limit curves.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and

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the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

For further details with respect to this action, see (1) the application for amendment dated August 15, 1975, (2) Amendment No. 6 to License No. DPR-51, with Change No. 6, and (3) the Commission's concurrently issued related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Arkansas Polytechnic College, Russellville, Arkansas 72801. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this *2nd day of October, 1975.*

FOR THE NUCLEAR REGULATORY COMMISSION  
Original Signed by  
Bartholomew C. Buckley

B. C. Buckley, Acting Chief  
Operating Reactors Branch #2  
Division of Reactor Licensing

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PRELIMINARY DETERMINATION

NOTICING OF PROPOSED LICENSING AMENDMENT

**Licensee:** Arkansas Power and Light Company (AP&L)

**Request for:** Arkansas Nuclear One - Unit 1 Tech Spec change to (1) modify the rod withdrawal limit curves to include limitations associated with maintaining potential ejected control rod worth within previously established limits (including following control rod interchange) and limitations associated with maintaining shutdown margin, (2) delete the separate specification on maximum inserted control rod worths, but include the limits and bases therefore in (1) above, (3) incorporate (cont'd on attached page)

**Request Date:** August 15, 1975

**Proposed Action:** ( ) Pre-notice Recommended  
( x ) Post-notice Recommended  
( ) Determination delayed pending completion of Safety Evaluation

**Basis for Decision:** This change is being made in response to our letter of March 26, 1975, informing AP&L of a deficiency in the Babcock & Wilcox (B&W) ejected rod worth calculational model. A new B&W model has been approved by the NRC staff. Changes similar to that requested by AP&L and based on the new model have been approved (without pre-notice) for Three Mile Island and Oconee 1,2,3 nuclear units. The requested change is a result of the application of a small refinement of a previously used calculational model and represents no relaxation of safety limits or limiting safety system settings. There is also no relaxation of a limiting condition for operation except to permit reactor operation for four hours with control rods in the newly defined "restricted region" based on the ejected rod accident. Such operation was previously found acceptable where the rod limits were based on a LOCA. This situation was considered in the Oconee package and found acceptable.

**CONCURRENCES:**

**DATE:**

1. WEConverse W.E. Converse 9/18/75
2. DLZiemann D. Ziemann 9/18/75
3. K. R. Collier Karl R. Collier 9/18/75
4. Blair Blair 9/24/75  
Office of Executive Legal Director

ATTACHMENT 1

Continuation of Request for:

an additional restriction on the regulating control rod positions prior to criticality to assure that the ejected rod worth does not exceed 1% delta k/k at hot zero power conditions, and (4) permit the rod withdrawal limit curves associated with ejected rod limits to be exceeded for a maximum period of four hours, provided that shutdown margin requirements are maintained and corrective measures are taken immediately to achieve a rod pattern consistent with the limit curves.