



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARKANSAS POWER & LIGHT COMPANY

DOCKET NO. 50-313

ARKANSAS NUCLEAR ONE - UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 20
License No. DPR-51

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Arkansas Power & Light Company (the licensee) dated January 13, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.c(2) of Facility Operating License No. DPR-51 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 20, are hereby incorporated in this license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 15, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 20

FACILITY OPERATING LICENSE NO. DPR-51

DOCKET NO. 50-313

Replace existing pages 76 and 77 of the Appendix A portion of the Technical Specifications with the attached revised pages bearing the same numbers. The changed areas on the revised pages are identified by a marginal line.

4.2 REACTOR COOLANT SYSTEM SURVEILLANCE

Applicability

Applies to the surveillance of the reactor coolant system pressure boundary.

Objective

To assure the continued integrity of the reactor coolant system pressure boundary.

Specification

- 4.2.1 Prior to initial unit operation, an ultrasonic test survey shall be made of reactor coolant system pressure boundary welds as required to establish preoperational integrity and baseline data for future inspections.
- 4.2.2 Post operational inspections of components shall be made in accordance with the methods and intervals indicated in IS-242 and IS-261 of Section XI of the ASME Boiler and Pressure Vessel Code, 1971, including 1972 Summer Addenda, except as follows:

<u>S-261 Item</u>	<u>Component</u>	<u>Exception</u>
1.4	Primary Nozzle to Vessel Welds	1 RC inlet nozzle to be inspected at approximately 3 1/3 years of the inspection interval. All four RC inlet nozzles to be inspected at or near the end of the inspection interval. At approximately 6 2/7 years of the inspection interval, both RC outlet nozzles will be inspected. At approximately 3 1/3 years of the inspection interval, one core flood nozzle will be inspected and one core flood nozzle will be inspected at or near the end of the inspection interval.
3.3	Safe Ends on Heat Exchanger	Not Applicable
4.1	Vessel Safe End Welds	Not Applicable
4.2	Valve Pressure Retaining Bolting Larger than 2"	Not Applicable
4.9	Integrally Welded Supports	Not Applicable
6.1	Valve Body Welds	Not Applicable
6.3	Valve to Safe End Welds	Not Applicable

<u>IS-261 Item</u>	<u>Component</u>	<u>Exception</u>
6.4	Bolting 2ø	Not Applicable
6.6	Integrally Welded Valve Supports	Not Applicable

- 4.2.3 The structural integrity of the reactor coolant system boundary shall be maintained at the level required by the original acceptance standards throughout the life of the station. Any evidence, as a result of the tests outlined in Table IS-261 of Section XI of the code, that defects have developed or grown, shall be investigated.
- 4.2.4 To ensure the structural integrity of the reactor internals throughout the life of the unit, the two sets of main internals bolts (connecting the core barrel to the core support shield and to the lower grid cylinder) shall remain in place and under tension. This will be verified by visual inspection to determine that the welded bolt locking caps remain in place. All locking caps will be inspected after hot functional testing and whenever the internals are removed from the vessel during a refueling or maintenance shutdown. The core barrel to core support shield caps will be inspected each refueling shutdown.
- 4.2.5 Sufficient records of each inspection shall be kept to allow comparison and evaluation of future inspections.
- 4.2.6 Complete surface and volumetric examination of the reactor coolant pump flywheels will be conducted coincident with refueling or maintenance shutdowns such that within a 10 year period after start-up all four reactor coolant pump flywheels will be examined.
- 4.2.7 Reactor vessel specimens shall be removed and examined, to determine changes in material properties, at specimen exposure ($E > 1Mey$) equivalent to 3, 9.5, 16 and 22.5 Effective Full Power Years (EFPY) of operation. This withdrawal schedule may be modified to coincide with those refueling outages or plant shutdowns, when the reactor head is removed, most closely approaching the withdrawal schedule. Results of these examinations shall be used to update Technical Specification 3.1.2. Specimens not subjected to destructive testing after the first 0.93 EFPY of Cycle 1 may be removed and stored during the remainder of Cycle 1, but shall be re-installed prior to Cycle 2.

Bases

The surveillance program has been developed to comply with Section XI of the ASME Boiler and Pressure Vessel Code Inservice Inspection of Nuclear Reactor Coolant Systems, 1974, including 1972 Summer Addenda edition.

The number of reactor vessel specimens and the frequencies for removing and testing these specimens are provided to assure compliance with the requirements of Appendix H to 10 CFR Part 50.