



NOV 22 2019

Michael J. Yox
Regulatory Affairs Director
Vogtle 3 & 4

7825 River Road
Waynesboro, GA 30830
706-848-6459 tel

Docket Nos.: 52-025
52-026

ND-19-1351
10 CFR 52.99(c)(3)

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555-0001

Southern Nuclear Operating Company
Vogtle Electric Generating Plant Unit 3 and Unit 4
Notice of Uncompleted ITAAC 225-days Prior to Initial Fuel Load
Item 3.3.00.02a.i.d [Index Number 763]

Ladies and Gentlemen:

Pursuant to 10 CFR 52.99(c)(3), Southern Nuclear Operating Company hereby notifies the NRC that as of November 15, 2019, Vogtle Electric Generating Plant (VEGP) Unit 3 and Unit 4 Uncompleted Inspection, Test, Analysis, and Acceptance Criteria (ITAAC) Item 3.3.00.02a.i.d [Index Number 763] has not been completed greater than 225-days prior to initial fuel load. The Enclosure describes the plan for completing ITAAC 3.3.00.02a.i.d [Index Number 763]. Southern Nuclear Operating Company will at a later date provide additional notifications for ITAAC that have not been completed 225-days prior to initial fuel load.

Southern Nuclear Operating Company (SNC) previously submitted Notice of Uncompleted ITAAC 225-days Prior to Initial Fuel Load for Item 3.3.00.02a.i.d [Index Number 763] ND-19-0042 [ML19030B726], dated Jan. 28, 2019. This resubmittal supersedes ND-19-0042 in its entirety.

This notification is informed by the guidance described in NEI-08-01, *Industry Guideline for the ITAAC Closure Process Under 10 CFR Part 52*, which was endorsed by the NRC in Regulatory Guide 1.215. In accordance with NEI 08-01, this notification includes ITAAC for which required inspections, tests, or analyses have not been performed or have been only partially completed. All ITAAC will be fully completed and all Section 52.99(c)(1) ITAAC Closure Notifications will be submitted to NRC to support the Commission finding that all acceptance criteria are met prior to plant operation, as required by 10 CFR 52.103(g).

This letter contains no new NRC regulatory commitments.

If there are any questions, please contact Tom Petrak at 706-848-1575.

Respectfully submitted,

Michael J. Yox
Regulatory Affairs Director Vogtle 3&4

U.S. Nuclear Regulatory Commission
ND-19-1351
Page 2 of 4

Enclosure: Vogtle Electric Generating Plant (VEGP) Unit 3 and Unit 4
Completion Plan for Uncompleted ITAAC 3.3.00.02a.i.d [Index Number 763]
MJY/GJL/sfr

To:

Southern Nuclear Operating Company/ Georgia Power Company

Mr. Peter P. Sena III (w/o enclosures)
Mr. D. L. McKinney (w/o enclosures)
Mr. M. D. Meier (w/o enclosures)
Mr. D. H. Jones (w/o enclosures)
Mr. G. Chick
Mr. M. Page
Mr. P. Martino
Mr. M. J. Yox
Mr. A. S. Parton
Ms. K. A. Roberts
Mr. T. G. Petrak
Mr. C. T. Defnall
Mr. C. E. Morrow
Mr. J. L. Hughes
Mr. S. Leighty
Ms. A. C. Chamberlain
Mr. J. C. Haswell
Document Services RTYPE: VND.LI.L06
File AR.01.02.06

cc:

Nuclear Regulatory Commission

Mr. W. Jones (w/o enclosures)
Mr. F. D. Brown
Mr. C. P. Patel
Mr. G. J. Khouri
Ms. S. E. Temple
Mr. N. D. Karlovich
Mr. A. Lerch
Mr. C. J. Even
Mr. B. J. Kemker
Ms. N. C. Coover
Mr. C. Welch
Mr. J. Gaslevic
Mr. V. Hall
Mr. G. Armstrong
Ms. T. Lamb
Mr. M. Webb
Mr. T. Fredette
Mr. C. Weber
Mr. S. Smith
Mr. C. Santos

Oglethorpe Power Corporation

Mr. R. B. Brinkman
Mr. E. Rasmussen

Municipal Electric Authority of Georgia

Mr. J. E. Fuller
Mr. S. M. Jackson

Dalton Utilities

Mr. T. Bundros

Westinghouse Electric Company, LLC

Dr. L. Oriani (w/o enclosures)
Mr. D. C. Durham (w/o enclosures)
Mr. M. M. Corletti
Ms. L. G. Iller
Mr. Z. S. Harper
Mr. J. L. Coward

Other

Mr. J. E. Hesler, Bechtel Power Corporation
Ms. L. Matis, Tetra Tech NUS, Inc.
Dr. W. R. Jacobs, Jr., Ph.D., GDS Associates, Inc.
Mr. S. Roetger, Georgia Public Service Commission
Ms. S. W. Kernizan, Georgia Public Service Commission
Mr. K. C. Greene, Troutman Sanders
Mr. S. Blanton, Balch Bingham

U.S. Nuclear Regulatory Commission
ND-19-1351 Enclosure
Page 1 of 14

**Southern Nuclear Operating Company
ND-19-1351
Enclosure**

**Vogtle Electric Generating Plant (VEGP) Unit 3 and Unit 4
Completion Plan for Uncompleted ITAAC Item 3.3.00.02a.i.d [Index No. 763]**

ITAAC Statement

Design Commitment

2.a) The nuclear island structures, including the critical sections listed in Table 3.3-7, are seismic Category I and are designed and constructed to withstand design basis loads as specified in the Design Description, without loss of structural integrity and the safety-related functions.

3.) Walls and floors of the nuclear island structures as defined on Table 3.3-1 except for designed openings or penetrations, provide shielding during normal operations.

Inspections/Tests/Analyses

i) An inspection of the nuclear island structures will be performed. Deviations from the design due to as-built conditions will be analyzed for the design basis loads, and for radiation shielding.

Acceptance Criteria

i.d) A report exists which reconciles deviations during construction, including Table 3.3-1 wall and floor thicknesses, and concludes that the as-built structures in the radiologically controlled area of the auxiliary building, including the critical sections, conform to the approved design and will withstand the design basis loads specified in the Design Description without loss of structural integrity or the safety-related functions, and without impacting compliance with the radiation protection licensing basis.

ITAAC Completion Description

Multiple ITAAC are performed to demonstrate that the nuclear island (NI) structures, including the critical sections listed in VEGP Unit 3&4 Combined License (COL) Appendix C (Reference 1 and 2) Table 3.3-7 (Attachment A), are seismic Category I and are designed and constructed to withstand design basis loads as specified in the VEGP Unit 3&4 COL Appendix C Section 3.3 Design Description, without loss of structural integrity and the safety-related functions. In addition, multiple ITAAC are performed on walls and floors of the NI structures as defined on Table 3.3-1 (Attachment B) except for designed openings or penetrations, that provide radiation shielding during normal operations.

The subject ITAAC verifies inspection of the as-built structures in the radiologically controlled area of the auxiliary building, including the critical sections and Table 3.3-1 wall and floor thicknesses, and reconciles deviations during construction to the approved design such that the as-built structures will withstand design basis loads without loss of structural integrity or the safety-related functions, and without impacting compliance with the radiation protection licensing basis.

Design bases loads are defined in VEGP Unit 3 COL Appendix C Section 3.3 as those loads associated with:

- Normal plant operation (including dead loads, live loads, lateral earth pressure loads, and equipment loads, including hydrodynamic loads, temperature and equipment vibration);
- External events (including rain, snow, flood, tornado, tornado generated missiles and earthquake); and
- Internal events (including flood, pipe rupture, equipment failure, and equipment failure generated missiles).

VEGP 3&4 Updated Final Safety Analysis Report (Reference 3), Section 3.7 "Seismic Design", Section 3.8 "Design of Category I Structures", and Appendix 3H "Auxiliary and Shield Building Critical Sections" describe the analyses for the design basis loads for the NI Structures. Section 3.8 specifies the applicable codes and standards governing the design, materials, fabrication, construction inspection and testing for the NI structures. Section 3.8 also describes the as-built design summary reports which document that the seismic Category I structures meet the specified acceptance criteria.

Radiation zone and equipment qualification requirements are met in accordance with VEGP 3&4 UFSAR Tier 2 design criteria including UFSAR Subsections 3.11.4 "Estimated Radiation and Chemical Environment," 3D.5.1.2 "Radiation Dose," and 12.3.2.1 "Shielding, Design Objectives".

The structures in the radiologically controlled area of the auxiliary building, including the critical sections, listed in Attachment A, and walls and floors of the NI structures as defined on Table 3.3-1 (Attachment B) except for designed openings or penetrations, provide radiation shielding during normal operations are constructed as designed and specified in the VEGP Unit 3 COL Appendix C Section 3.3 Design Description to withstand the Design Description design basis loads without loss of structural integrity and the safety-related functions, and without impacting compliance with the radiation protection licensing basis.

The structures in the radiologically controlled area of the auxiliary building, including the critical sections, listed in Attachment A, and walls and floors of the NI structures as defined on Table 3.3-1 (Attachment B), except for designed openings or penetrations, which provide radiation shielding during normal operations are inspected during construction to verify the as-built structures conform to the specified design, codes and standards. Construction identified structural deviations are documented, evaluated, and reconciled by engineering to confirm the structures' ability to withstand design basis loads without impacting compliance with the radiation protection licensing basis. The As-Built Summary Reports (References 4 through 9) exist and document the reconciliation of NI structural deviations identified during construction and conclude that the as-built structures in the radiologically controlled area of the auxiliary building, including the critical sections and walls and floors of the NI structures as defined on Table 3.3-1 except for designed openings or penetrations, will withstand the design basis loads specified in the Design Description without loss of structural integrity or the safety-related functions, and without impacting compliance with the radiation protection licensing basis.

Unit 3 & 4 Principle Closure Documents (References 4 through 9) are available for NRC inspection as part of the Unit 3 & 4 ITAAC 3.3.00.02a.i.d Completion Packages (References 10 and 11)

List of ITAAC Findings

In accordance with plant procedures for ITAAC completion, Southern Nuclear Operating Company (SNC) performed a review of all findings pertaining to the subject ITAAC and associated corrective actions. This review found twenty (20) NRC findings associated with this ITAAC.

1. 05200025/2016001-01 (Closed – ML19135A691)
2. 05200026/2016001-01 (Closed – ML19135A691)
3. 05200025/2015002-02 (Closed – ML19045A614)
4. 05200025/2015002-01 (Closed – ML16032A554)
5. 05200026/2015002-01 (Closed – ML16032A554)
6. 05200025/2015001-01 (Closed – ML15124A857)
7. 05200025/2014005-01 (Closed – ML16032A554)
8. 05200025/2014002-01 (Closed – ML14218A213)
9. 05200025/2013004-01 (Closed – ML14087A320)
10. 05200025/2013003-01 (Closed – ML13207A241)
11. 05200025/2013002-01 (Closed – ML13207A241)
12. 05200025/2012004-01 (Closed – ML13312A316)
13. 05200025/2012004-02 (Closed – ML14024A594)
14. 05200025/2012008-01 (Closed – ML12319A458)
15. 05200025/2012008-02 (Closed – ML12319A458)
16. 99901448/2014-201-02 (Closed – ML17226A340)
17. 99901449/2014-201-02 (Closed – ML17226A340)
18. 99901425/2014-201-01 (Closed – ML18101A168)
19. 99901419/2012-201-03 (Closed – ML18131A260)
20. 99901409/2011-201-03 (Closed – ML18186A573)

The ITAAC completion review is documented in the ITAAC Completion Packages for ITAAC 3.3.00.02a.i.d Unit 3 and Unit 4 (Reference 10 and 11) and are available for NRC review.

References (available for NRC inspection)

1. VEGP Unit 3 COL Appendix C, Amendment 167
2. VEGP Unit 4 COL Appendix C, Amendment 165
3. VEGP 3&4 UFSAR, Revision 8.1
4. As-Built Summary Report for Unit 3 Auxiliary Building (except CA20 and Basemat), SV3-AAA-BBB-###
5. As-Built Summary Report for Unit 4 Auxiliary Building (except CA20 and Basemat), SV4-AAA-BBB-###
6. As-Built Summary Report for Unit 3 CA20, SV3-CCC-DDD-###
7. As-Built Summary Report for Unit 4 CA20, SV4-CCC-DDD-###
8. As-Built Summary Report for Unit 3 Nuclear Island Basemat, SV3-EEE-FFF-###
9. As-Built Summary Report for Unit 4 Nuclear Island Basemat, SV4-EEE-FFF-###
10. 3.3.00.02a.i.d-U3-CP-Rev0, ITAAC Completion Package
11. 3.3.00.02a.i.d-U4-CP-Rev0, ITAAC Completion Package
12. NEI 08-01, "Industry Guideline for the ITAAC Closure Process Under 10 CFR Part 52"

Attachment A: Excerpt of COL Appendix C Table 3.3-7

Table 3.3-7
Nuclear Island Critical Structural Sections
<u>Auxiliary Building</u> South wall of auxiliary Building (column line 1), elevation 66'-6" to elevation 180'-0" Divider Wall Between the spent fuel pool and the fuel transfer canal.
<u>Nuclear Island Basemat Below Auxiliary Building</u> Bay between reference column lines 1 and 2, and K2 and N

Attachment B: Excerpt of COL Appendix C Table 3.3-1

Table 3.3-1 Definition of Wall Thicknesses for Nuclear Island Buildings, Turbine Building, and Annex Building⁽¹⁾				
Wall or Section Description	Column Lines⁽⁷⁾	Floor Elevation or Elevation Range⁽⁷⁾⁽⁸⁾	Concrete Thickness⁽²⁾⁽³⁾⁽⁴⁾⁽⁵⁾⁽⁹⁾⁽¹⁸⁾	Applicable Radiation Shielding Wall (Yes/No)
Auxiliary Building Walls/Floors Radiologically Controlled⁽¹⁵⁾				
Column Line 1 wall	From I to N	From 66'-6" to 100'-0"	3'-0"	No
Column Line 1 wall	From I to 5'-7" west of K-2	From 100'-0" to 109'-3"	2'-3"	Yes
Column Line 1 wall	From 5'-7" west of K-2 to N	From 100'-0" to 109'-3"	3'-0"	Yes
Column Line 1 wall	From I to 5'-6" east of L-2	From 109'-3" to 180'-0"	2'-3"	Yes
Column Line 1 wall	From 5'-6" east of L-2 to N	From 109'-3" to 125'-0"	3'-0"	Yes
Column Line 1 wall	From 5'-6" east of L-2 to N	From 125'-0" to 180'-0"	2'-3"	Yes
Column Line 2 wall	From I to K-2	From 66'-6" to 135'-3"	2'-6"	Yes
Column Line 2 wall	From K-2 to L-2	From 66'-6" to 135'-3"	5'-0"	Yes
Column Line 2 wall	From L-2 to N	From 82'-6" to 135'-3"	2'-6"	Yes
Column Line 2 wall	From I to J-1	From 135'-3" to 153'-0"	2'-0"	Yes
Column Line 3 wall	From J-1 to J-2	From 66'-6" to 82'-6"	2'-6"	Yes
Column Line 3 wall	From J-1 to J-2	From 100'-0" to 135'-3"	2'-6"	Yes
Column Line 3 wall	From J-2 to K-2	From 66'-6" to 135'-3"	2'-6"	Yes
Column Line 3 wall	From K-2 to L-2	From 66'-6" to 92'-8 1/2"	2'-6"	Yes
Column Line 4 wall	From I to J-1	From 66'-6" to 135'-3"	2'-6"	Yes
Column Line 4 wall	From J-1 to J-2	From 66'-6" to 92'-6"	2'-6"	Yes

Attachment B: Excerpt of COL Appendix C Table 3.3-1

Table 3.3-1 Definition of Wall Thicknesses for Nuclear Island Buildings, Turbine Building, and Annex Building⁽¹⁾				
Wall or Section Description	Column Lines⁽⁷⁾	Floor Elevation or Elevation Range⁽⁷⁾⁽⁸⁾	Concrete Thickness⁽²⁾⁽³⁾⁽⁴⁾⁽⁵⁾⁽⁹⁾⁽¹⁸⁾	Applicable Radiation Shielding Wall (Yes/No)
Auxiliary Building Walls/Floors Radiologically Controlled⁽¹⁵⁾				
Column Line 4 wall	From J-1 to J-2	From 107'-2" to 135'-3"	2'-6"	Yes
Column Line 4 wall	From J-2 to K-2	From 66'-6" to 135'-3"	2'-6"	Yes
Column Line 4 wall	From I to intersection with shield building wall	From 135'-3" to 180'-0"	2'-0"	Yes
Column Line 5 wall	From I to shield building; with opening east of J-1 (below 107'-2" floor).	From 66'-6" to 160'-6"	2'-0"	Yes
Wall, 17'-3" north of Column Line 7	From I to 8' east of J-1	From 66'-6" to 82'-6"	2'-0"	Yes
Wall, 10'-6" south of Column Line 7.3	From I to 5'-6" east of J-1	From 66'-6" to 82'-6"	2'-0"	Yes
Wall, 10'-6" south of Column Line 7.3	From I to just east of J	From 82'-6" to 100'-0"	2'-0"	Yes
Column Line I wall	From 1 to 4	From 66'-6" to 100'-0"	3'-0"	No
Column Line I wall	From 4 to 16'-0" south of 5	From 66'-6" to 107'-2"	3'-0"	No
Column Line I wall	From 16'-0" south of 5 to 5	From 66'-6" to 105'-0"	3'-0"	No
Column Line I wall	From 5 to 7.3	From 66'-6" to 100'-0"	3'-0"	No
Column Line I wall	From 1 to 3	From 100'-0" to roof	2'-0"	Yes
Column Line I wall	From 3 to 4	From 100'-0" to roof	2'-0"	Yes

Attachment B: Excerpt of COL Appendix C Table 3.3-1

Table 3.3-1 Definition of Wall Thicknesses for Nuclear Island Buildings, Turbine Building, and Annex Building⁽¹⁾				
Wall or Section Description	Column Lines⁽⁷⁾	Floor Elevation or Elevation Range⁽⁷⁾⁽⁸⁾	Concrete Thickness⁽²⁾⁽³⁾⁽⁴⁾⁽⁵⁾⁽⁹⁾⁽¹⁸⁾	Applicable Radiation Shielding Wall (Yes/No)
Auxiliary Building Walls/Floors Radiologically Controlled⁽¹⁵⁾				
Column Line I wall	From 4 to 16'-0" south of 5	From 107'-2" to roof	2'-0"	No
Column Line I wall	From 16'-0" south of 5 to 5	From 105'-0" to roof	2'-0"	No
Column Line J-1 wall	From 1 to 2	From 82'-6" to 100'-0"	2'-0"	Yes
Column Line J-1 wall	From 2 to 4	From 66'-6" to 135'-3"	2'-6"	Yes
Column Line J-1 wall	From 2 to 4	From 135'-3" to 153'-0"	2'-0"	Yes
Column Line J-1 wall	From 4 to shield building	From 66'-6" to 107'-2"	2'-0"	Yes
Column Line J-2 wall	From 2 to 4	From 66'-6" to 135'-3"	2'-6"	Yes
Column Line J-2 wall	From 4 to intersection with shield building wall	From 66'-6" to 135'-3"	2'-0"	Yes
Column Line K-2 wall	From 2 to 4	From 66'-6" to 135'-3"	4'-9"	Yes
Column Line L-2 wall	From 2 to 4	From 66'-6" to 135'-3"	4'-0"	Yes
Column Line N wall	From 1 to 2	From 66'-6" to 100'-0"	3'-0"	No
Column Line N wall	From 1 to 12'-9" north of 1	From 100'-0" to 125'-0"	3'-9"	Yes
Column Line N wall	From 1 to 12'-9" north of 1	From 125'-0" to 135'-3"	2'-0"	Yes
Column Line N wall	From 12'-9" north of 1 to 2	From 100'-0" to 118'-2 1/2"	3'-0"	Yes
Column Line N wall	From 12'-9" north of 1 to 2	From 118'-2 1/2" to 135'-3"	2'-0"	Yes

Attachment B: Excerpt of COL Appendix C Table 3.3-1

Table 3.3-1 Definition of Wall Thicknesses for Nuclear Island Buildings, Turbine Building, and Annex Building⁽¹⁾				
Wall or Section Description	Column Lines⁽⁷⁾	Floor Elevation or Elevation Range⁽⁷⁾⁽⁸⁾	Concrete Thickness⁽²⁾⁽³⁾⁽⁴⁾⁽⁵⁾⁽⁹⁾⁽¹⁸⁾	Applicable Radiation Shielding Wall (Yes/No)
Auxiliary Building Walls/Floors Radiologically Controlled⁽¹⁵⁾				
Column Line N wall	From 2 to 4 (or to shield building)	From 66'-6" to 98'-1"	5'-6"	No
Column Line N wall	From 2 to 4 (or to shield building)	From 98'-1" to 135'-3"	5'-6" ⁽¹¹⁾	Yes
Column Line N wall	From 1 to 4 (or to shield building)	From 135'-3" to 180'-0"	2'-0"	Yes
Labyrinth Wall between Col. Line 3 and 4 and J-1 to 5'-2" from J-2	Not Applicable	From 82'-6" to 92'-6"	2'-6"	Yes
N-S Shield Wall (low wall)	5'-7" west of column line K-2 extending 16'-0" from column line 1 north	From 100'-0" to 110'-0"	2'-6"	Yes
N-S Shield Wall	2'-9" east of column line L-2 extending 12'-9" from column line 1 north	From 100'-0" to 109'-3"	3'-0"	Yes
N-S Shield Wall	2'-9" east of column line L-2 extending 12'-9" from column line 1 north	From 109'-3" to 125'-0"	2'-9"	Yes
E-W Shield Wall	10'-0" north of column line 1 extending 13'-3" from column line N east	From 100'-0" to 125'-0"	2'-9"	Yes
Auxiliary Area Basemat	From 1-7.3 and I-N, excluding shield building	From 60'-6" to 66'-6"	6'-0"	No

Attachment B: Excerpt of COL Appendix C Table 3.3-1

Table 3.3-1 Definition of Wall Thicknesses for Nuclear Island Buildings, Turbine Building, and Annex Building⁽¹⁾				
Wall or Section Description	Column Lines⁽⁷⁾	Floor Elevation or Elevation Range⁽⁷⁾⁽⁸⁾	Concrete Thickness⁽²⁾⁽³⁾⁽⁴⁾⁽⁵⁾⁽⁹⁾⁽¹⁸⁾	Applicable Radiation Shielding Wall (Yes/No)
Auxiliary Building Walls/Floors Radiologically Controlled⁽¹⁵⁾				
Floor	From 1 to 2 and I to N	82'-6"	2'-0"	Yes
Floor	From 2 to 4 and J-1 to J-2	82'-6"	2'-0"	Yes
Floor	From 4 north to the shield building ending 17'-4" south of column line 5 and J-1 to J-2	82'-6"	0'-9"	Yes
Pipe Chase Floor	From 2 north to the shield building ending 17'-4" south of column line 5 and J-1 to J-2	92'-6"	2'-0"	Yes
Floor	From 2 to 3 and J-2 to K-2	90'-3"	3'-0"	Yes
Floor	From 3 to 4 and J-2 to K-2	92'-6"	2'-0"	Yes
Floor	From 4 to 7.3 and I to J-1	82'-6"	2'-0"	Yes
Floor	From 1 to 2 and I to N	100'-0"	3'-0"	Yes
Floor	From 2 to 4 and K-2 to L-2	92'-8 1/2"	3'-2 1/2"	Yes
Floor	From I to J-2 and 4 to shield building and vertical wall 17'-0" south of column line 5	107'-2"	2'-0"	Yes

Attachment B: Excerpt of COL Appendix C Table 3.3-1

Table 3.3-1 Definition of Wall Thicknesses for Nuclear Island Buildings, Turbine Building, and Annex Building⁽¹⁾				
Wall or Section Description	Column Lines⁽⁷⁾	Floor Elevation or Elevation Range⁽⁷⁾⁽⁸⁾	Concrete Thickness⁽²⁾⁽³⁾⁽⁴⁾⁽⁵⁾⁽⁹⁾⁽¹⁸⁾	Applicable Radiation Shielding Wall (Yes/No)
Auxiliary Building Walls/Floors Radiologically Controlled⁽¹⁵⁾				
Floor	From I to shield building wall and from intersecting vertical wall before column line 5 to column line 5	105'-0"	0'-9"	Yes
Floor	From column line 1 to 10'-0" north of column line 1 and from 2'-9" east of column line L-2 to N	125'-0"	3'-0"	Yes
Floor	From 12'-9" north of column line 1 to 2 and from 2'-9" east of column line L-2 to N	118'-2 1/2"	2'-0"	Yes
Floor	From 3 to 4 and J-2 to K-2	117'-6"	2'-0"	Yes
Floor	From 2 to 4 and I to J-1	153'-0"	1'-3"	Yes
Roof	From 1 to 4 and I to N	180'-0" to 180'-9"	1'-3"	Yes
Floor	From 4 to 16'-0" south of column line 5 and from I to intersection with shield building wall	135'-3"	0'-9"	Yes

Attachment B: Excerpt of COL Appendix C Table 3.3-1

Table 3.3-1 Definition of Wall Thicknesses for Nuclear Island Buildings, Turbine Building, and Annex Building⁽¹⁾				
Wall or Section Description	Column Lines⁽⁷⁾	Floor Elevation or Elevation Range⁽⁷⁾⁽⁸⁾	Concrete Thickness⁽²⁾⁽³⁾⁽⁴⁾⁽⁵⁾⁽⁹⁾⁽¹⁸⁾	Applicable Radiation Shielding Wall (Yes/No)
Auxiliary Building Walls/Floors Radiologically Controlled⁽¹⁵⁾				
Floor	From 16'-0" south of column line 5 to column line 5 and from I to intersection with shield building wall	133'-0"	0'-9"	Yes

Attachment B: Excerpt of COL Appendix C Table 3.3-1

1. The column lines and floor elevations are identified and included on Figures 3.3-1 through 3.3-13.
2. These wall (and floor) thicknesses have a construction tolerance of ± 1 inch, except as noted and for exterior walls below grade where the tolerance is +12 inches, - 1 inch. These tolerances are not applicable to the nuclear island basemat.
3. For walls that are part of structural modules, the concrete thickness also includes the steel face plates. Where faceplates with a nominal thickness of 0.5 inches are used in the construction of the wall modules, the wall thicknesses in this column apply. Where faceplates thicker than the nominal 0.5 inches are used in the construction of the structural wall modules, the wall thicknesses in the area of the thicker faceplates are greater than indicated in this column by the amount of faceplate thickness increase over the nominal 0.5 inches. Overlay plates are not considered part of the faceplates, and thus are not considered in the wall thicknesses identified in this column.
4. For floors with steel surface plates, the concrete thickness also includes the plate thickness.
5. Where a wall (or a floor) has openings, the concrete thickness does not apply at the opening.
6. N/A to ITAAC 3.3.00.02a.i.d.
7. The Wall or Section Description, Column Line information, and Floor Elevation or Elevation Ranges are provided as reference points to define the general location. The concrete thickness of an item intersecting other walls, roofs or floors at a designated location (e.g., column line) is not intended to be measured to the stated column line, but only to the point where the intersection occurs.
8. Where applicable, the upper wall portions extend to their associated roofs, which may vary in elevation, e.g., sloped roofs.
9. From one wall/floor section to another, the concrete thickness transitions from one thickness to another, consistent with the configurations in Figures 3.3-1 through 3.3-14.
10. N/A to ITAAC 3.3.00.02a.i.d.
11. This wall thickness has a tolerance of +4 inches, -1 inch above grade.
12. N/A to ITAAC 3.3.00.02a.i.d.
13. N/A to ITAAC 3.3.00.02a.i.d.
14. N/A to ITAAC 3.3.00.02a.i.d.
15. Reconciliation of construction deviations in the nuclear island structures from the thickness and tolerances specified in this table is included in the reconciliation reports, and demonstrate that the as-built structures will withstand design basis loads without loss of structural integrity or safety functions and without impacting compliance with the radiation protection licensing basis, such as GDC 19, established radiological zoning and equipment qualification in accordance with ITAAC 3.3.00.02a.i.a, 3.3.00.02a.i.d, 3.3.00.02a.i.d, or 3.3.00.02a.i.d.
16. N/A to ITAAC 3.3.00.02a.i.d.
17. N/A to ITAAC 3.3.00.02a.i.d.
18. Nonconformances from the thicknesses and tolerances specified in Table 3.3-1 (i.e. out of tolerance conditions) are addressed under the 10 CFR Part 50, Appendix B process and subsequently are screened in accordance with the 10 CFR Part 52, Appendix D, Section VIII process, to ensure that the licensing basis is adequately maintained. Construction deviations will continue to be assessed against the licensing basis requirements and will be addressed in accordance with licensee procedures and regulatory requirements and, if applicable, a license amendment will be obtained prior to implementation of the change.