UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

ARKANSAS POWER & LIGHT COMPANY

DOCKET NO. 50-313

ARKANSAS NUCLEAR ONE - UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 7 License No. DPR-51

1. The Nuclear Regulatory Commission (the Commission) has found that:

- A. The application for amendment by Arkansas Power & Light Company (the licensee) dated October 7, 1975 (as it relates to tendon surveillance), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
- B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
- C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
- D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.
- Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.c(2) of Facility License No. DPR-51 is hereby amended to read as follows:



"(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 7 ."

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by Bartholomew C. Buckley

Dennis L. Ziemann, Chief Operating Reactors Branch #2 Division of Reactor Licensing

Attachment: Change No. 7 to the Technical Specifications

Date of Issuance:

NOV 2 0 1975

ATTACIMENT TO LICENSE AMENDMENT NO. 7 CHANGE NO. 7 TO THE TECHNICAL SPECIFICATIONS FACILITY OPERATING LICENSE NO. DPR-51

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DOCKET NO. 50-513

Replace existing page 36 of the Technical Specifications with the attached revised page 36. The changed area on the revised page is reflected by a marginal line. Also, page 85 is enclosed as a matter of convenience in updating the Technical Specifications. There are no changes on this page.

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Form AEC-318 (Rev. 9-53) AECM 0240

A U. S. GOVERNMENT PRINTING OFFICE: 1974-526-185

4.4.2 Structural Integrity

Applicability

Applies to the structural integrity of the reactor building.

Objective

To define the structural integrity of the reactor building.

Specification

4.4.2.1 Tendon Surveillance

For the tendon surveillance program, to be conducted over the life of the unit, twenty-one tendons shall be selected for periodic inspection for symptoms of material deterioration or force reduction. The surveillance tendons shall consist of ten hoop tendons, at least three in each of the three 240° sectors of the reactor building; five vertical tendons located at approximately equally spaced intervals; and six dome tendons, two in each of the three groups of dome tendons.

4.4.2.1.1 Lift-Off

Lift-off readings shall be taken for all 21 surveillance tendons.

4.4.2.1.2

Wire Inspection and Testing

A minimum of three surveillance tendons, one from each of the hoop, vertical, and dome families, shall be relaxed and one wire from each relaxed tendon shall be removed as a sample and visually inspected for corrosion or pitting. In addition, the applicable anchor assemblies shall be inspected for deleterious conditions, such as corrosion, cracks, missing wires and off size button heads. Tensile and elongation tests shall also be performed on a minimum of three specimens taken from the ends and middle of each of the wires. The specimens shall be the maximum length acceptable for the test apparatus to be used and shall include areas representative of significant corrosion or pitting.

After the wire removal, the tendons shall be retensioned to the stress level measured at the lift-off reading (and changes in shim thicknesses shall be recorded) and then checked by a final lift-off reading. The tendon elongation during retensioning shall be measured. Should the inspection of one of the wires reveal any significant physical change (pitting or loss of area), additional wires shall be removed from the applicable surveillance tendons and inspected to determine the extent and cause change. The sheathing filler will be sampled and inspected for changes in physical appearance.

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4.4.2.2 Inspection Intervals and Reports

The inspection intervals, reasured from the date of the initial structural test, shall be one year, three years, five years, and every five years thereafter or as modified based on experience. Tendon surveillance may be conducted during reactor operation provided design conditions regarding loss of adjacent tendons are satisfied at all times.

A quantitative analytical report covering results of each inspection shall be submitted (required by Technical Specification 6.7) and shall especially address the following conditions, should they develop:

(1) Broken wires.

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- (2) The force-time trend line for any tendon, when extrapolated, that extends beyond either the upper or lower bounds of the predicted design band.
- (3) Unexpected changes in tendon conditions or sheathing filler properties.
- 4.4.2.3 End Anchorage Concrete Surveillance
 - A. The end anchorages of the surveillance tendons and adjacent concrete surface will be inspected.
 - B. The inspection interval will be one-half year and one year after the structural integrity test.
 - C. The selected inspection location shall include:
 - Four (4) locations on one buttress (hoop tendon anchorage)
 - (2) Two (2) locations on the top of the ring girder (vertical tendon anchorage).
 - (3) One (1) location on the ring girder (dome tendon anchorage).