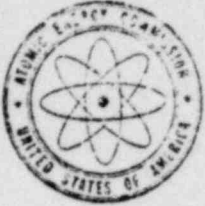


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UNITED STATES
ATOMIC ENERGY COMMISSION

~~XXXXXXXXXXXXXXXXXXXX~~

REGION II - SUITE 818

230 PEACHTREE STREET, NORTHWEST
ATLANTA, GEORGIA 30303

TELEPHONE: (404) 526-4503

DIRECTORATE OF REGULATORY OPERATIONS

MAY 23 1972

In Reply Refer To:
RO:II:VLB
50-313/72-2
50-368/72-2

Arkansas Power and Light Company
Attn: Mr. J. D. Phillips
Vice President and Chief Engineer
Sixth and Pine Streets
Pine Bluff, Arkansas 71601

Gentlemen:

This refers to the inspection conducted by Messrs. Brownlee, Crossman, Swan and Jape on March 28-30, 1972, of activities authorized by AEC Construction Permit No. CPPR-57 for the Arkansas Nuclear One, Unit 1 facility and of your quality assurance program for work to be performed under an exemption from the requirements of 10 CFR 50.10 for the Arkansas Nuclear One, Unit 2 facility. Our findings were discussed by Mr. Brownlee with Messrs. Moore, Bland, and Bean of your staff, and Messrs. Zampieri and Sly of Bechtel Engineering Corporation at the conclusion of the inspection.

Areas examined during the inspection included the installation of electrical and instrumentation components and cable, piping systems, major primary pressure boundary valves, and the post-tensioning system; containment liner erection for Unit 2, control of drawings, and control of measuring and test equipment onsite.

Within the scope of this inspection, there were no items of noncompliance identified.

We have examined the actions you have taken with regard to uniformity of weld transitions between piping and valves or fittings identified in our letter of January 24, 1972, and ultrasonic testing of the top end of the pressurizer vessel identified in our letter of January 28, 1972. In addition, the inspectors examined the actions you have taken with regard to our letter of December 9, 1971, concerning a scram breaker failure at another facility. We have no further questions concerning these items.

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MAY 23 1972

Arkansas Power and
Light Company

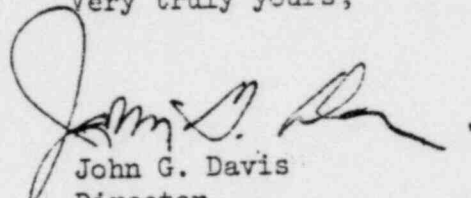
- 2 -

From discussions with a member of your staff, it is our understanding that you will determine the applicability to your facility and conduct an evaluation of the failure of a reactor in-core instrument guide tube which occurred at another facility. We will examine this item further on a subsequent inspection.

As discussed with members of your staff, our records indicate that two items remained unresolved and will require additional review by our inspectors. The items were the main steam relief valve manifold stress evaluation identified in our letter of December 10, 1971, and the tendon wire stressing washer, identified in our letter of January 24, 1972.

A reply to this letter is not necessary; however, should you have any questions concerning this letter, you may communicate directly with this office.

Very truly yours,



John G. Davis
Director