

FS&EB ACTION CONTROL FORM

JUL 17 1973

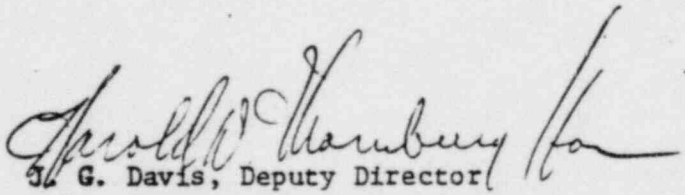
A. Action Code 131  
Name of Licensee and Facility Arkansas Power & Light (Arkansas Nuclear One - Unit 1)  
Docket No. or License No. 50-313  
Title CDR Evaluation and Followup  
Origin CDR Date Rec'd 7/10/73

B. FS&EB Branch Coordinator:  
Bryan \_\_\_\_\_ Dreher \_\_\_\_\_  
Ellis \_\_\_\_\_ Paulus X  
Completion Requested by \_\_\_\_\_

C. Action Requested of:  
ADREMP \_\_\_\_\_ M&PPOB \_\_\_\_\_ EPB \_\_\_\_\_ RPB \_\_\_\_\_ ADCO \_\_\_\_\_  
OB \_\_\_\_\_ CB \_\_\_\_\_ TAB \_\_\_\_\_ OOE \_\_\_\_\_ Region II  
Date Requested 7/12/73 Completion Requested by \_\_\_\_\_  
Reference Letter dated July 2, 1973 from J. D. Phillips to D. F. Knuth

D. Action Requested  
In accordance with PI 0600/6, "Construction Deficiency Reporting", the Arkansas Power & Light Company (Arkansas Nuclear One - Unit 1) deficiency report of 7/2/73 covering thin walled valves is being assigned to Region II for evaluation of the technical adequacy of the corrective action and the final resolution of the deficiency.

E. Date Action Completed \_\_\_\_\_  
Close-out (Date & Method) \_\_\_\_\_  
Comments: If completion date is not consistent with your work schedule, please let us know.

  
G. Davis, Deputy Director  
for Field Operations  
Directorate of Regulatory Operations

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HELPING BUILD ARKANSAS

## ARKANSAS POWER & LIGHT COMPANY

8TH AVENUE AND PINE STREET • PINE BLUFF, ARKANSAS 71601 • (501) 534-1330

July 2, 1973

Mr. D. F. Knuth  
Director of Regulatory Operations  
United States Atomic Energy Commission

SUBJECT: ARKANSAS POWER & LIGHT COMPANY  
ARKANSAS NUCLEAR ONE - UNIT 1  
DOCKET NO. 50-313  
SIGNIFICANT DEFICIENCY REPORT

Dear Mr. Knuth:

The following is a report submitted as required by 10 CFR 50.55(e) concerning a deficiency identified to the Principal Reactor Inspector assigned to Arkansas Nuclear One, Unit 1, by telephone on June 1, 1973, relating to nine (9) twelve inch and fourteen inch check and gate valves procured from Velan Engineering Company.

During the conduct of AP&L's valve wall thickness verification program of primary pressure boundary valves, six (6) nuclear Class I check valves were found that did not meet the requirements of Table 452.1 of "Draft ASME Code for Pumps and Valves for Nuclear Power" and three (3) Class II gate valves were found that do not meet minimum wall requirements of ANSI B16.5.

Subsequent investigation indicates that the three (3) Class II gate valves have acceptable wall thickness because for these valves MSS-SP-66 and ASME Code Case 1329 are applicable.

We anticipate that repair work for the six (6) Class I check valves will be performed at the nuclear plant site by the valve manufacturer. The manufacturer (Velan) is presently obtaining ASME Code authorization for these repairs. Velan's welding procedure has been received and approved and an additional repair procedure is under review. This work will not affect the scheduled completion of the preoperational testing program.

Very truly yours,

J. D. PHILLIPS  
Senior Vice President

JDP:NAM:mb



Mr. D. F. Knuth

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July 2, 1973

cc: Mr. Norman C. Moseley, Director  
United States Atomic Energy Commission  
Region II  
230 Peachtree Street, N.W., Suite 818  
Atlanta, Georgia 30303