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R. S. Boyd
R. L. Tedesco
R. R. Powell
T. Engelhardt, OGC
H. Steele (2)

November 25, 1970

Docket No. 50-346

Ace-Federal Reporters, Inc.
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Gentlemen:

Twenty copies each of the following documents are enclosed for inclusion in the transcript of the hearing scheduled to be held December 8, 1970, in Port Clinton, Ohio on the application for construction permit for the Davis-Besse Nuclear Power Station:

1. Summary Statement by the Division of Reactor Licensing (Davis-Besse Nuclear Power Station) dated December 8, 1970.
2. Professional Qualifications - R. L. Tedesco.
3. Professional Qualifications - R. R. Powell

Ten additional copies of the above documents will be furnished ACE-Federal at the hearing.

Very truly yours,

Original signed by
H. Steele

Helen Steele, Licensing Assistant
Division of Reactor Licensing

Enclosures:
As stated above

OFFICE ▶	DRL: BWR-2						State
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PREHEARING CONFERENCE STATEMENT

DAVIS-BESSE NUCLEAR POWER STATION

DOCKET NO. 50-346

November 23, 1970

On August 1, 1969, The Toledo Edison Company, and The Cleveland Electric Illuminating Company (applicants) applied to the Atomic Energy Commission for a construction permit and facility license for the proposed Davis-Besse Nuclear Power Station. The plant will be located on the southwestern shore of Lake Erie in Ottawa County, Ohio, approximately 21 miles east of Toledo, Ohio.

The Davis-Besse Nuclear Power Station will be owned and financed by the applicants, The Toledo Edison Company, and the Cleveland Electric Illuminating Company, as tenants-in-common. The Toledo Edison Company will be responsible for the design, construction and operation of the station.

The plant will use a two-loop pressurized water reactor. The nuclear steam supply system will be supplied by Babcock & Wilcox. The balance of the plant will be designed and constructed by the Bechtel Corporation. The reactor is designed to operate at an initial power level of 2633 thermal megawatts (MWt) with an expected ultimate power level of 2772 MWt. The design of the engineered safety features, including the containment structure, emergency core cooling systems, and calculations of consequences of certain postulated accidents have been analyzed by the applicants and evaluated by

us for the higher power level of 2772 MWt. However, evaluation of thermal, hydraulic and nuclear core design characteristics was performed for a power level of 2633 MWt. Before operation at any power level above 2633 MWt is authorized, the applicants must provide supporting analyses and data for our evaluation to assure that the core can be operated safely at the higher power level.

The facility incorporates features which are the same as those used on other plants for which the Commission has issued construction permits. These plants include Prairie Island Nuclear Power Plant, Three Mile Island Nuclear Power Plants Units 1 and 2, Arkansas Nuclear One, and Oconee Unit 1.

Several consultants, Environmental Science Services Administration, U.S. Geological Survey, Fish and Wildlife Service, and John A. Blume & Associates, were used in connection with our evaluation of the facility. Reports from these agencies are included as appendices to our Safety Evaluation dated November 2, 1970.

We have calculated the consequences resulting from releases of radioactivity following certain highly unlikely accidents assumed for purposes of evaluation and have concluded that the potential radiological doses would not exceed the guidelines set forth in 10 CFR Part 100 of the Commission's regulations.

The Advisory Committee on Reactor Safeguards (ACRS) has also conducted an independent review of this plant and has made comments and recommendations

in its report of August 20, 1970, to Chairman Seaborg. We have considered each of the recommendations and will be guided by all of them in our continuing review of the Davis-Besse facility.

The ACRS letter concludes that with due consideration to the various items mentioned therein, "the Davis-Besse Nuclear Power Station can be constructed with reasonable assurance that it can be operated without undue risk to the health and safety of the public."

On September 10, 1970, the Commission granted an exemption from the provisions of Section 50.10(b) of 10 CFR Part 50 to permit the applicants to perform subgrade concrete and reinforcing steel placement for the shield building and auxiliary building.

We have concluded, on the basis of our review of the application and eleven amendments, that appropriate findings in support of issuance of the construction permit can be made on each of the issues set forth in the Notice of Hearing in this proceeding.