



Commonwealth Edison
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November 1, 1989

Dr. Thomas E. Murley, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: Dresden Nuclear Power Station Unit 1
Supplement to Proposed Amendment to
Reflect Non-Operating Status
NRC Docket No. 50-010

References: (See Attachment A)

Dear Dr. Murley:

In Reference (a), Commonwealth Edison (CECo) submitted a proposed amendment to change the Dresden Station Unit 1 Technical Specifications to reflect its non-operating status.

Subsequently, the Commission issued Amendment No. 36 to License DPR-2 which modified the license to a possess-but-not-operate status. At this time it was noted that further action on CECo's Technical Specification change request would await submittal of CECo's decommissioning plan and Environmental Report which supported these changes.

In References (d) and (e), the NRC requested additional information concerning the above mentioned documents including the proposed Dresden Unit 1 Technical Specifications.

In response to the requested additional information (References (f) and (g)) CECo committed to submit a revision to the proposed Technical Specifications submitted in Reference (a).

The following attachments provide the revision to the proposed Dresden Unit 1 Technical Specifications. Attachment B contains background information and justification for the proposed change. Attachment C contains the proposed change to the Technical Specifications. The proposed change has been reviewed and approved by both On-Site and Off-Site Review in accordance with Commonwealth Edison Company procedures. This amendment request has been evaluated in accordance with 10 CFR 50.90(c) and it was determined that no significant hazards consideration exists. That evaluation is documented in Attachment D.

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November 1, 1989

Commonwealth Edison is notifying the State of Illinois of our supplemental application for amendment by transmitting a copy of this letter and its attachments to the designated State Official.

Please direct any questions you may have to this office.

Very truly yours,

Wayne E Morgan

W.E. Morgan
Nuclear Licensing Administrator

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Attachments

cc: P.E. Erickson - Project Manager, NRR
B.L. Siegel - Project Manager, NRR
S.G. DuPont - Senior Resident Inspector - Dresden
Office of Nuclear Facility Safety - IDNS
A.B. Davis - Regional Administrator, Region III

ATTACHMENT A

- References (a): J.R. Wojnarowski letter to H.R. Denton, "Dresden Unit 1 Proposed License Amendment to Reflect Non-Operating Status" dated January 7, 1986.
- (b): W.E. Morgan letter to U.S. NRC, "Dresden Unit 1 Decommissioning Plan, Environmental Report, and Revised Tech Specs" dated December 22, 1987.
- (c): W.E. Morgan letter to U.S. NRC, "Dresden Unit 1 Decommissioning Documents", dated April 26, 1988.
- (d): P.B. Erickson letter to W.E. Morgan, "Dresden Unit 1 Decommissioning Plan and Technical Specifications - Request for Additional Information", dated September 27, 1988.
- (e): P.B. Erickson letter to W.E. Morgan, "Dresden Unit 1 Emergency Plan and Technical Specifications - Request for Additional Information", dated January 7, 1989.
- (f): W.E. Morgan letter to U.S. NRC, "Response to Request for Additional Information Decommissioning Plan, Technical Specifications and Emergency Plans", dated March 27, 1989.
- (g): W.E. Morgan letter to U.S. NRC, "Response to Request for Additional Information - Dresden Unit 1 - Emergency Plan", dated April 10, 1989.

ATTACHMENT B

TECHNICAL SPECIFICATION CHANGE REQUEST

DRESDEN STATION UNIT 1

BACKGROUND:

On January 7, 1986 (Reference (a)), CECO submitted a proposal to amend Facility Operating License DPR-2 and Appendix A of the Technical Specifications, to reflect the non-operating status of Dresden Unit 1. On July 23, 1986, the Commission issued Amendment No. 36 to License DPR-2. This amendment modified the license to a possess-but-not-operate status. At this time the Staff stated that further NRC action on the Technical Specification changes would await Edison's submittal of a safety analysis and/or Decommissioning Plan to support the proposed changes. On December 22, 1987 (Reference (b)), Edison submitted the Decommissioning Program Plan. A revised Decommissioning Plan was submitted April 26, 1988 (Reference (c)) in conjunction with the Dresden Unit 1 Environmental Report.

On September 27, 1988 (Reference (d)) and January 7, 1989 (Reference (e)), the Commission requested additional information on the Decommissioning Plan, the Environmental Report, and the proposed Unit 1 Technical Specifications for the SAFSTOR period. Dresden Station responded to these requests for additional information in References (f) and (g). These responses committed Dresden to submit a revision to the original proposed Technical Specifications which had been submitted. The purpose of this document is the submittal of the proposed Technical specification changes.

ATTACHMENT C

PROPOSED CHANGES TO THE TECHNICAL SPECIFICATIONS FOR LICENSE DPR-2

- Following are a summary of changes initiated due to changes in the Units 2/3 Technical Specifications:

On June 30, 1989, the NRC issued amendments to the Units 2 and 3 Technical Specifications. These amendments brought the Technical Specifications into compliance with the guidance in NRC Generic Letter 88-12, which permits the removal of detailed fire protection requirements when certain other programmatic requirements have been included. Dresden has placed the detailed fire protection program requirements in the FSAR and in the Dresden Administrative Technical Requirements (DATR). These changes have also been included in the Unit 1 Technical Specifications.

On April 26, 1989, the NRC issued amendments to the Units 2 and 3 Technical Specifications which removed offsite and on-site organizational charts in accordance with the guidance in Generic Letter 88-06. In addition, other organizational title changes were approved. The Organizational Charts will be maintained in the CECO Quality Assurance Manual and/or the Management Plan for Nuclear Operations. This change has been incorporated into Section 2.D of the Amended Facility License (Fire Protection).

- Section-by-section description of changes between the currently approved Technical Specification and the revised proposed Technical Specifications:

1.0 Definitions

The definition section has been modified by deleting all items pertaining to operating reactors and retaining only those frequently used terms applicable to the SAFSTOR mode. Two definitions were added for the Offsite Dose Calculation Manual (ODCM) and the Process Control Program (PCP).

2.0 Safety Limits and Limiting Safety System Settings - Deleted

These limits and settings were operational in nature and do not apply to a unit in SAFSTOR.

3.0 Limiting Conditions for Required Equipment

The following Limiting Conditions for Required Equipment (LCRE), which were formerly LCOs in the operational Technical Specifications, have been retained with various modifications. All other operational LCOs have been deleted, such as the reactor protection system, protective instrumentation, reactivity control, standby liquid control, core and containment cooling, primary system boundary, containment system, high energy piping, and fuel cladding integrity.

These deletions are justified, since: (1) fuel is not allowed in the vessel, (2) the reactor cannot be operated, and (3) no heat is generated in the primary system. In addition, the primary and secondary systems have been chemically cleaned; thereby removing most of the loose activation radioactivity. Therefore, containment is no longer required.

3.8 Radioactive Materials

A. Airborne Effluents

Unit 2 and 3 Technical Specification Amendments No. 83 and 77, approved in 1984, complied with the guidelines of 10 CFR 50 Appendix I, Numerical Guides for Design Objectives and LCO to Meet the Criterion ALARA for Radioactive Material in Light-Water-cooled Nuclear Power Reactor Effluents. Section 3.8/4.8, Radioactive Materials, of the Unit 2 and 3 Technical Specifications are identical and specify dose rate limits in unrestricted areas at or beyond the Dresden Site boundary from all gaseous effluent sources within the site. The proposed Unit 1 airborne effluent Technical Specifications section states that Unit 1 gaseous effluents shall be added to the total site effluents and be included in the site dose rate as calculated from specified samples and following the methods prescribed in the approved Dresden ODCM.

The proposed Unit 1 Technical Specifications will continue to specify continuous main chimney monitoring. This is because even though no iodine will be released from the shutdown Unit 1, the potential still exists for small releases from Units 2 and 3 through the adjacent Unit 1 turbine building. The minimum analysis frequency of iodine has been decreased to monthly from weekly. All other analysis frequencies remained unchanged. Surveillance requirements of the main chimney continuous monitor have been modified to be consistent with those specified in the Units 2 and 3 Technical Specifications.

B. Liquid Effluents

The proposed Unit 1 Technical Specifications state that radioactive effluents shall not be released from Unit 1 storage tanks directly to the environment; however, release may be made through the Unit 2/3 Radwaste system. Continuous monitoring of the Unit 1 service water discharge is required in the proposed Technical Specifications. These revised monitoring requirements are essentially equivalent to the currently approved Technical Specifications that applied during periods when Unit 1 has been in the shutdown condition. Since discharges are made only on an infrequent basis, surveillances are required prior to performing and once per day during planned discharges instead of on a daily basis.

C. Radioactive Waste Storage

The waste storage limits proposed are identical to those in the present Unit 1 Technical Specification. Unit 1 chemical cleaning waste is presently stored in the Chemical Cleaning Facility. The solidification and shipping of these wastes will be handled under the Dresden Nuclear Power Station (DNPS) Process Control Program (PCP). The U.S. Ecology topical report for this portion of the PCP is under separate staff review. The DNPS PCP is referenced in the Unit 1 Technical Specification.

D. General

This section remains the same as the existing Unit 1 Technical Specifications with two exceptions: (1) a commitment to the environmental monitoring program is made in this section and (2) the portions of the existing Technical Specifications that specified effluent concentration limits has been deleted.

The Bases have also been modified as appropriate.

3.9 Auxilliary Electrical Systems

Since Unit 1 is shutdown with fuel in storage, Dresden proposes to delete the auxilliary electrical Technical Specifications. In the existing Technical Specifications, this section included the operation requirements for: (1) two 138 KV lines, (2) an operable diesel generator with starting batteries, (3) the 34.5 KV substation, (4) two 480 volt buses, and (5) the 125 volt station batteries. These systems were required to be operable before the reactor was taken critical. Unit 1 is in the SAFSTOR conditions, as related in the above section. The worst case radiological release is less than one percent of 10 CFR Part 100 guidelines, as shown in the following section. Therefore, there is no safety related need for electrical power to contain and monitor the Unit 1 radioactive materials. The stored irradiated fuel, the greatest potential hazard, has decayed until the heat load no longer requires external cooling. This is evidenced by the fuel storage pool cooling system being out of service since 1983. Only occasional water additions are required to maintain the water shielding level. The water additions are necessary to replace evaporation losses.

All applicable Unit 1 electrical systems will be maintained and, where necessary, tested with the exception of: (1) the main generator, (2) Transformer numbers 1 and 11, and (3) the emergency diesel generator and associated equipment. This maintenance and testing will be done using the existing preventive maintenance and surveillance program. These programs have been in effect since the Unit 1 shutdown in 1978.

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3.10 Fuel Handling and Storage

This new section replaces a former LCO titled Refueling. All requirements formerly pertaining to refueling, such as, interlocks, nuclear instrumentation, control rod (CR) and CR drive maintenance have been deleted from the Technical Specifications. A new item specifies that nuclear fuel will not be loaded into the reactor.

The new section continues to specify daily checks of the Fuel Storage Pool minimum water level. In addition to the daily checks, a pool level float actuates an alarm in the radwaste building and also in the Unit 1 Control Room. The area radiation monitor in the Fuel Handling Building would also give an alarm in the Unit 1 Control Room if pool level decreased significantly.

The worst foreseeable accident with radiological consequences at Unit 1 in SAFSTOR is considered to be the rupture of all 683 fuel assemblies stored in the Fuel Handling Building. Dresden has had an evaluation performed of such an accident (Reference (g)) to determine the worst case exposure to: (1) personnel in the Fuel Handling Building and (2) to the public at the nearest unrestricted area boundary or exclusion area boundary (EAB). The analysis indicated a Kr-85 concentration in the Fuel Handling Building of 5.181 curies per cubic meter giving a total skin dose rate of 1085 rem per hour in the worst case accident.

For the offsite release, the same assumptions were used but no credit was taken for the Fuel Handling Building or the ventilation system (a ground level release over a two hour period). The whole body dose at the nearest EAB (475 meters) was estimated to be 0.016 rem and the worst skin dose 1.7 rem.

Standard Review Plan, NUREG 0800, in Section 15.7.5, Spent Fuel Cask Drop Accidents, states that plant siting and dose mitigating engineered safety feature systems are acceptable if the whole body dose at the EAB is well within the exposure guideline values of 10 CFR Part 100. The term, well within, is defined as 25 percent of the Part 100 values or six rem for whole body doses. The estimated whole body dose at the nearest Dresden EAB in this worst case accident is less than one percent of the six rem specified in 10 CFR Part 100.

Dresden has not included Technical Specification limitations on heavy loads over the fuel or Fuel Handling Building ventilation controls. Because of the low level of significance of this improbable worst case release, Dresden has included procedural control of Fuel Handling Building doors during fuel handling and will include heavy load procedural controls when fuel cask loading procedures are developed.

The proposed SAFSTOR Technical Specifications specify fuel pool water quality limits with monthly sampling and analysis for specific conductivity, chlorides, and pH. These water quality limits will be required to be met once initial pool cleanup is complete (Reference 6). Dresden Administrative Procedure, DAP 1-3, Administrative Control Program for Dresden Unit 1, dated March 1989, implements fuel pool water quality limits which are more restrictive than Technical Specifications. A weekly sample and analysis of the pool water is scheduled in Procedure DCP 1020-1, Chemistry Surveillance Schedule.

3.12 Fire Protection Systems

As previously stated, Dresden has moved the fire protection program from the Unit 2 and 3 Technical Specification and incorporated the program into the DATR. The Unit 1 Technical Specifications submittal proposes to do the same for Unit 1 as in the Units 2 and 3 Technical Specifications. Dresden commits in the Unit 1 license to implement and maintain in effect the provisions of the fire protection program in the DATR. Changes in the approved program without prior approval of the NRC are permitted only if the changes would not adversely affect the fire protection capability.

The proposed Unit 1 fire protection program for the DATR remains essentially unchanged from the existing Technical Specification. The fire detection instrumentation requirement has been deleted because these detectors were located only in the following three areas: Unit 1 Core Spray Pump Area, Unit 1 Diesel Generator Area, and the Unit 1 Primary Feed Pump Area. The detector installation was never completed in Unit 1. The installation is no longer needed in SAFSTOR since the above safety equipment is no longer operational. Therefore, fire detection in these areas is no longer necessary.

Since the Unit 1 Fire Suppression Water System is an integral part of the Dresden Station Fire Suppression System, the Unit 2 and 3 DATR program also includes the Unit 1 diesel fire pump surveillance requirements. Wet pipe sprinklers have been specified for the Unit 1 Diesel Fire Pump area which brings the DATR into agreement with the UFSAR. The sprinkler system is included in Table 3.1-2 of the DATR surveillance program. The proposed Unit 1 Fire Pump Diesel Engine surveillance requirements, including starting batteries and charger, remain the same as the Operational Technical Specifications.

The requirement for the Unit 1 CO₂ Suppression Systems has been deleted from the proposed DATR fire protection program. This is because the two systems: (1) in the Unit 1 Emergency Diesel Generator Bay Tank Area and (2) in the Feedwater Pump Area, are no longer needed since this equipment is no longer operational.

The proposed surveillance requirements for the Unit 1 Fire Hose Stations remain the same as Units 2 and 3; however, seven hose stations in the Unit 1 Turbine Building are no longer included in the proposed DATR program, leaving only two stations in the Turbine Building and one in the Crib House. These are included in DATR Table 3.10-5. Although the seven stations were removed from the DATR program, they still remain operational and are included in the Dresden Station fire hose surveillance program.

The requirement for Unit 1 penetration fire barriers has been deleted because safety related equipment needed to safely operate or shutdown the reactor is no longer necessary.

Modifications under way to consolidate needed Unit 1 instrumentation and controls into the Unit 2 and 3 Control Room will provide a new fire and security wall between the Unit 2 and 3 Control Room and a new adjacent office area formerly the Unit 1 Control Room. All penetrations falling within the Unit 2 and 3 Control Room will be included in the Unit 2 and 3 fire barrier program. All existing Unit 1 penetrations into the new office area will be permanently sealed. All of the other existing Unit 1 penetration fire barriers will be left in place and maintained.

5.0 Design Features

Only the site description and the fuel storage design features have been retained; the reactor, reactor vessel, containment, and seismic design features have been deleted.

6.0 Administrative Controls

The organization, review, investigation, and audit sections of the proposed Technical Specification have been updated to bring them into agreement with the organization, position titles, and requirements that have been previously approved in License Amendments 97 and 92 for Units 2 and 3. Subsequent to these amendments, on April 14, 1989, the NRC approved Unit 2 and 3 amendments to remove the Corporate and Dresden Station organization charts from the Technical Specifications in accordance with the Commission's recommendations of Generic Letter 88-06. In addition, other changes such as position titles, the distribution of onsite reviews, and other typographical and editorial changes were approved. The Unit 1 Administrative Technical Specification will be consistent with the Unit 2 and 3 Technical Specifications in that organization charts have been removed and the CECO organization is now depicted in the CECO Quality Assurance Manual or the Management Plan for Nuclear Operations. Staff position titles and responsibilities have been changed to agree with the Unit 2 and 3 Technical Specifications.

Some of the Unit 1 administrative requirements have been modified to take into consideration the shutdown condition. These are as follows:

1. The Offsite Dose Calculation Manual and the Dresden Nuclear Power Station Process Control Program (PCP) have been included in the Unit 1 Technical Specifications to update them even though most radioactive wastes will be processed through Units 2 and 3.
2. The following procedure areas were eliminated because they are no longer needed:
 - a. Refueling Procedures.
 - b. Primary System Leaks.
 - c. Abnormal Reactor Changes.
 - d. Safe Shutdown Procedures.
3. The actions to be taken if Safety Limits are exceeded were removed.
4. The following records requirements were eliminated since they are no longer applicable.
 - a. Radioactive shipment records since all waste processing is carried out through Units 2 and 3.
 - b. New fuel records.
 - c. Transient or operational cycling of certain equipment.
 - d. Inservice Inspections.
 - e. Environmental Qualifications of Equipment.
5. The reporting requirements have been changed by elimination of the Startup Report, Monthly Operating Report and all Special reports except the radioactive source leak testing, which was retained.

The proposed Unit 1 Technical Specification format has been changed to be compatible with the Unit 2 and 3 formats.

Note: Technical Specification section numbers have been changed to mirror the Unit 2/3 Technical Specification section numbers as closely as possible.

(1) Page ii and iii

Table of Contents changes to reflect changes in the balance of the Technical Specification.

- (2) Page 1v
List of Tables and List of Figures have been modified to reflect changes in the balance of the Technical Specifications.
- (3) Page 1.0-2
New page inserted to add definition of the ODCM and PCP.
- (4) Page 1/2.1-1
New page inserted to indicate sections that have been deleted.
- (5) Page 3/4.1-1
New page inserted to indicate sections that have been deleted.
- (6) Page 3/4.8-1
Added reference to the ODCM in section 3.8.A.2. Changed plant chimney to main chimney in Section 4.8.A.
- (7) Page 3/4.8-3
Changed plant chimney to main chimney. Added reference to the ODCM in section 4.8.H. Changed "may" to "shall" in section 4.8.E.1.b.
- (8) Page B.3/4.8-6
Added reference to the ODCM in section B.4.8.H.
- (9) Pages 3/4.9-1 and B 3/4.9-2
Deleted Auxiliary Electrical Systems Section
- (10) Pages 3/4.10-1 and 3/4.10-2
Deleted Section 3.10-D: Control Rod and Control Rod Drive Maintenance - Added Section on Fuel Storage Pool Water Quality.
- (11) Pages 3/4.12-1 and 3/4.12-4
Deleted Fire Protection Section
- (12) Page 5-1
Deleted Sections 5.2: Reactor; 5.3: Reactor Vessel; and 5.4: Containment

- (13) Pages 6-1 through 6-25
Replaced entire administrative controls section with a modified Unit 2 Administrative controls section.
- (14) Page 6-6
Added frequency to the audits in section 6.1.G.1.a.(4) and (7). Added item 6.1.G.1.a.(13): Changes to the Decommissioning Plan.
- (15) Page 6-13
Changed 6.2.A.1 to reflect the shutdown status of Unit 1. Deleted 6.2.A.2.
- (16) Page 6-14
Deleted 6.2.A.9.
- (17) Page 6-15
Changed wording in Section 6.2.D.2. Deleted section 6.4, Action to be Taken in the Event of a Safety Limit is Exceeded.
- (18) Page 6-16
Deleted 6.5.A.1. Changed wording in Section 6.5.A.2.
- (19) Page 6-17
Deleted section 6.5.A.10: regarding records of transient cycling.
Deleted section 6.5.A.14: regarding Environmental Qualification.
Deleted section 6.6.A.1 on the Startup Report.
Deleted Section 6.6.A.3: Monthly Operating Report.
- (20) Page 6-20
Deleted section 6.7: Environmental Qualification
- (21) Page 6-21
Deleted most of Table 6.6.1: Special Reports.

Procedures affected by the Technical Specification Change:

DOA 9P3-D11, Service Water and Cooling Water Discharge HI RAD ACT

DOA 1700-2, High Radioactivity - Service Water Discharge

DOA 6900-1, 125 VDC (Operating Battery) Systems Failure

DOS 6900 series, Unit 1 125 VDC Storage Battery

DFP 800 series, Unit 1 Fuel Procedures

DIS 1700-2, Service Water Radiation Monitor Calibration and Functional Test

DIS 4100-1, Fire Pump and Bearing Lube Water Pump Discharge Pressure Gauges
Calibration

DIS 5200-1, Diesel Fuel Oil Storage Tank Level Indication Calibration

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-10

AMENDED FACILITY LICENSE

Amendment No.

License No. DPR-2

1. The Atomic Energy Commission (the Commission) has found that:
 - A. The application, as amended, for license filed by the Commonwealth Edison Company (the licensee) complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I and all required notifications to other agencies or bodies have been duly made;
 - B. The facility will be maintained in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amended license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the rules and regulations of the Commission;
 - D. The licensee is technically and financially qualified to engage in the activities authorized by this amended license in accordance with the rules and regulations of the Commission;
 - E. The licensee has satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements," of the Commission regulations;
 - F. The issuance of this amended license will not be inimical to the common defense and security or to the health and safety of the public and does not involve a significant hazards consideration; and
 - G. The receipt, possession, and use of byproduct, source and special nuclear materials as authorized by this license will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40, and 70, including Sections 30.33, 40.32, 70.23, and 70.31.

2. Facility License No. DPR-2 issued to the Commonwealth Edison Company is hereby amended in its entirety to read as follows:
- A. This license applies to the Dresden Nuclear Power Station Unit 1 dual-cycle, boiling water reactor (herein the facility), owned by the Commonwealth Edison Company. The facility is located in Grundy County, Illinois, and is described in the application attested to on May 31, 1955, and subsequent amendments thereto, including the amendment dated May 17, 1973.
- B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses the Commonwealth Edison Company:
- (1) Pursuant to Section 104b of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities," to possess, use and maintain but not to operate the facility at the designated location in Grundy County, Illinois, in accordance with the procedures and limitations set forth in this license;
 - (2) Pursuant to the Act and 10 CFR Part 70, to possess at any time special nuclear materials, not including plutonium, as reactor fuel, in accordance with limitations for storage as described in Hazards Summary Report (Final Safety Analysis Report), as supplemented and amended as of September 3, 1976;
 - (3) Pursuant to the Act and 10 CFR Part 70, to possess at any time up to 6631 grams of plutonium utilized in previous operations of the facility.
 - (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess at any time any byproduct, source and special nuclear materials as sealed neutron sources for reactor startup and as fission detectors in amounts required, and to receive, possess and use sealed sources for reactor instrumentation and radiation monitoring equipment calibration.
 - (5) Pursuant to the Act and 10 CFR 30, 40, and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear materials without restriction to chemical or physical form, for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components;
 - (6) Pursuant to the Act and 10 CFR Parts 30 and 70, to receive, possess, but not separate, such byproduct and special nuclear materials as may have been produced by the operation of the facility."

- C. The licensee shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70 is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below.

(1) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. , are hereby incorporated in the license. The licensee shall maintain the facility in accordance with the Technical Specifications.

D. FIRE PROTECTION

The licensee fire protection program is outlined in the Dresden Administrative Procedures. The limiting conditions for operation and surveillance requirements are outlined in the Dresden Administrative Technical Requirements. The above provisions are subject to the following:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the fire protection capability.

3. This amended license is effective as of the date of issuance and shall expire at midnight, March 29, 2029.

FOR THE ATOMIC ENERGY COMMISSION

Original signed by

Karl R. Goller
Assistant Director for Operating
Reactors
Directorate of Licensing

Date of Issuance:

WP+4144

Appendix A

to

Amended Facility License DPR-2

Technical Specifications and Bases For

Dresden Nuclear Power Station Unit 1

Grundy County, Illinois

Commonwealth Edison Company

Docket No. 50-10

Date: