

November 15, 1989

Docket No. 50-395

Mr. O. S. Bradham
Vice President, Nuclear Operations
South Carolina Electric & Gas Company
Virgil C. Summer Nuclear Station
P.O. Box 88
Jenkinsville, South Carolina 29065

Dear Mr. Bradham:

SUBJECT: VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1, - PRESSURE-TEMPERATURE LIMITS RELATING OF GENERIC LETTER 88-11. (TAC 71554)

By letter dated January 12, 1989, you responded to Generic Letter 88-11, "NRC Position on Radiation Embrittlement of Reactor Vessel Materials and Its Effect on Plant Operations," which requires licensees to use Regulatory Guide 1.99, Rev. 2 to calculate the nil-ductility reference temperature of reactor vessel beltline materials. The reference temperature relates to pressure-temperature limits in the V. C. Summer Nuclear Station Unit No. 1 Technical Specification 3.4.

The NRC staff has completed the review of the submittal with assistance from the Idaho National Engineering Laboratory. Enclosed is our Safety Evaluation (SE). Based on our evaluation, we conclude that the current pressure/temperature limits for the reactor coolant system for heatup, cooldown, leak test, and criticality are acceptable through 8 EFPY (Effective full power years). Since this SE only covers operation through 8 EFPY, you should submit a revised curve to address operation past the 8 EFPY prior to reaching the 8 EFPY. It is our understanding that you will have the benefits of the data from the capsule removed during the March 1990 refueling outage.

This SE closes out our work on Multiplant Action (A023) TAC No. 71554 for Summer.

Original Signed By:

J. J. Hayes, Jr. Project Manager
Project Directorate II-1
Division of Reactor Projects I/II

Enclosure:
as stated

cc w/enclosure:
See next page

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South Carolina Electric & Gas Company

Virgil C. Summer Nuclear Station

cc:

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