



Consumers  
Power

**POWERING  
MICHIGAN'S PROGRESS**

Big Rock Point Nuclear Plant, 10269 US-31 North, Charlevoix, MI 49720

November 2, 1989

Thomas W Elward  
Plant Manager

Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Dear Sir:

Enclosed please find the statistical data for the Big Rock Point Nuclear Plant covering the period of October 1, 1989 through October 31, 1989.

Sincerely,

T W Elward  
Plant Manager

Enclosures

cc: Administrator Region III, Nuclear Regulatory Commission  
DRHahn, Department of Public Health  
RCallen, Michigan Public Service Commission  
SHall, Michigan Department of Labor  
PDKrippner, American Nuclear Insurers  
INPO Record Center  
NRC Resident Inspector  
Document Control, Big Rock Point, 740/22\*35\*10  
DPHoffman, P24-117B  
KWBerry, P24-614B  
File

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NUCLEAR OPERATION DEPARTMENT  
Unit Shutdowns and Power Reductions

Report Month October, 1989	Docket Number 55-150	Unit Big Rock Point Plant	Date November 3, 1989	Completed by JRJohnston	Telephone (616) 547-6537 ext 223
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Number	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	License Event Report Number	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause and Corrective Action to Prevent Recurrence
89-10	10/09/89 thru 10/12/89	F	0 hrs	A	4	-----	-----	-----	The plant load was reduced to allow the clearing of the turbine thrust bearing wear indicator alarm. Auxiliary pressure sensing instrumentation was installed to where "mills" of thrust bearing clearance can be correlated to hydraulic nozzle pressure. It is thought that the wear indication is not a true reading since it is known that the thrust bearing wear indication reference point was moved (or lost) when the number 1 bearing cap was recently retightened. We are closely monitoring the indicated wear instrumentation and carrying reduced plant load so that the annunciator is not "in".
89-11	10/21/89 thru 10/23/89	F	0 hrs	A	4	-----	-----	-----	The plant load was reduced to allow repairs to the steam and water leaks on the turbine. It was successful in restricting or sealing leaks on the HP casing. The water leak is on the IP to LP crossover line and will require more planning to make repairs to it. The unit was returned to normal power until plans are completed to make temporary repairs.
89-12	10/25/89 thru 10/27/89	F	0 hrs	A	4	-----	-----	-----	The plant load was reduced to make temporary repairs to the expansion joint on the west side crossover pipe. The leak was found to be split on the second (from the bottom) serration in the north quadrant. After repairs are completed the plant will return to normal power.
89-13	10/27/89	F	100.2	A	1	-----	-----	-----	While raising power following repairs to the turbine crossover piping, it was discovered that the turbine bypass valve would not open. The valve was isolated and troubleshooting is in progress. The control circuit and components have checked out okay and the problems appear to be in either the actuating cylinder or the valve itself. The unit will be returned to normal power after repairs are completed.

<sup>1</sup>F = Forced  
S = Scheduled

<sup>2</sup>Reason:  
A = Equipment Failure (Explain)  
B = Maintenance of Test  
C = Refueling  
D = Regulatory Restriction  
E = Operator Training and License Examination  
F = Administrative  
G = Operational Error (Explain)  
H = Other (Explain)

<sup>3</sup>Method:  
1 = Manual  
2 = Manual Scram  
3 = Automatic Scram  
4 = Other (Explain)

<sup>4</sup>Exhibit G = Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>Exhibit I = Same Source

GREYBOOK OPERATING DATA REPORT

DOCKET NO. 50-155 DATE: 11 / 2 / 89  
 BY: JR JOHNSTON  
 PHONE: 616-547-6537, EXT 223

OPERATING STATUS

1. UNIT NAME: BIG ROC: POINT NUCLEAR PLANT

2. REPORTING PERIOD: 10 / 89

3. LICENSED THERMAL POWER (MWT): 240

4. NAMEPLATE RATING (GROSS MWE): 75

5. DESIGN ELECTRICAL RATING (NET MWE): 72

6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 71.0

7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 67.0

8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THRU 7) SINCE LAST REPORT, GIVE REASONS:

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE):

10. REASONS FOR RESTRICTIONS, IF ANY:

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	745.0	7296.0	233121.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	642.0	5525.9	167754.7
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
14. HOURS GENERATOR ON-LINE	644.8	5497.9	164906.0
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	129875.0	1063449.0	31260213.0
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	41220.0	343368.0	9920851.0
18. NET ELECTRICAL ENERGY GENERATED (MWH)	39053.9	324851.0	9382530.9
19. UNIT SERVICE FACTOR	86.6%	75.4%	70.7%
20. UNIT AVAILABILITY FACTOR	86.6%	75.4%	70.7%
21. UNIT CAPACITY FACTOR (USING MDC NET)	78.2%	66.5%	59.8%
22. UNIT CAPACITY FACTOR (USING DER NET)	72.8%	61.8%	55.9%
23. UNIT FORCED OUTAGE RATE	13.4%	2.4%	12.7%

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, & DURATION OF EACH):

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: NOVEMBER 4, 1989.

DAY AVERAGE DAILY POWER (MWT) (MWEN) (10/89) - CYCLE 24

DAY	AVERAGE DAILY POWER (MWT)	(MWEN)
1	214.25	64.54
2	214.25	64.42
3	216.67	65.50
4	218.67	64.84
5	216.00	65.03
6	215.04	64.86
7	215.54	64.66
8	215.00	64.58
9	199.58	60.35
10	210.12	63.15
11	194.92	59.42
12	211.42	63.56
13	216.17	64.87
14	216.29	65.00
15	216.96	64.92
16	215.62	64.78
17	218.25	65.66
18	218.12	65.46
19	216.46	65.29
20	216.25	65.26
21	165.17	49.08
22	159.17	48.66
23	211.75	64.08
24	214.83	64.75
25	204.50	61.48
26	85.96	24.97
27	94.50	28.05
28	0.0	0.0
29	0.0	0.0
30	0.0	0.0
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Refueling Information Request

1. Facility name: Big Rock Point Plant
2. Scheduled date for next refueling shutdown: September, 1990
3. Scheduled date for restart following shutdown: October, 1990
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? No

If yes, explain:

If no, has the reload fuel design and core configuration been reviewed by Plant Safety Review Committee to determine whether any unreviewed safety questions as associated with the core reload (Reference 10 CFR, Section 50.59)? No

If no review has taken place, when is it scheduled? Next outage

5. Scheduled date(s) for submittal of proposed licensing action and supporting information:
6. Important licensing considerations associated with refueling, eg, new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design new operating procedures:
7. Number of fuel assemblies in: core 84; spent fuel storage pool 254; new fuel storage 0.
8. Present licensed spent fuel storage capacity: 441  
Size of any increase in licensed storage capacity that has been requested or is planned (in number of fuel assemblies): 0
9. Projected date of last refueling that can be discharged to spent fuel pool assuming the present license capacity: 1995