



EDISON PLAZA
300 MADISON AVENUE
TOLEDO, OHIO 43652-0001

October 11, 1989
KB89-00486

Docket No. 50-346
License No. NPF-3

Document Control Desk
U. S. Nuclear Regulatory Commission
7920 Norfolk Avenue
Bethesda, MD 20555

Gentlemen:

Monthly Operating Report, September 1989
Davis-Besse Nuclear Power Station Unit 1

Enclosed are ten copies of the Monthly Operating Report for Davis-Besse Nuclear Power Station Unit No. 1 for the month of September 1989.

If you have any questions, please contact Bilal Sarsour at (419) 321-7384.

Very truly yours,

Louis F. Storz
Plant Manager
Davis-Besse Nuclear Power Station

BMS/mjb

Enclosures

cc: Mr. A. Bert Davis
Regional Administrator, Region III

Mr. Paul Byron
NRC Resident Inspector

Mr. T. V. Wambach
NRC Senior Project Manager

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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-346

UNIT Davis-Besse #1

DATE October 11, 1989

COMPLETED BY Bilal Sarsour

TELEPHONE (419) 321-7384

MONTH September 1989

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>865</u>
2	<u>866</u>
3	<u>768</u>
4	<u>796</u>
5	<u>873</u>
6	<u>870</u>
7	<u>867</u>
8	<u>862</u>
9	<u>859</u>
10	<u>874</u>
11	<u>875</u>
12	<u>875</u>
13	<u>877</u>
14	<u>876</u>
15	<u>879</u>
16	<u>875</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>859</u>
18	<u>877</u>
19	<u>875</u>
20	<u>874</u>
21	<u>874</u>
22	<u>803</u>
23	<u>195</u>
24	<u>506</u>
25	<u>883</u>
26	<u>881</u>
27	<u>882</u>
28	<u>879</u>
29	<u>878</u>
30	<u>877</u>
31	<u></u>

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

OPERATING DATA REPORT

DOCKET NO. 50-346
 DATE October 11, 1989
 COMPLETED BY Bilal Sarsour
 TELEPHONE (419) 321-7384

OPERATING STATUS

1. Unit Name: Davis-Besse Unit #1
2. Reporting Period: September 1989
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 925
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 918
7. Maximum Dependable Capacity (Net MWe): 874

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____

10. Reasons For Restrictions, If Any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720.0	6,551.0	97,920.0
12. Number Of Hours Reactor Was Critical	720.0	6,338.1	51,945.6
13. Reactor Reserve Shutdown Hours	0.0	89.0	5,393.7
14. Hours Generator On-Line	720.0	6,297.6	49,991.4
15. Unit Reserve Shutdown Hours	0.0	0.0	1,732.5
16. Gross Thermal Energy Generated (MWH)	1,892,722	16,991,252	118,860,641
17. Gross Electrical Energy Generated (MWH)	629,351	5,664,796	39,305,980
18. Net Electrical Energy Generated (MWH)	597,625	5,376,095	36,841,147
19. Unit Service Factor	100.0	96.1	51.1
20. Unit Availability Factor	100.0	96.1	52.8
21. Unit Capacity Factor (Using MDC Net)	95.0	93.9	43.0
22. Unit Capacity Factor (Using DER Net)	91.6	90.6	41.5
23. Unit Forced Outage Rate	0.0	2.4	29.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

Refueling - Start February 1, 1990 - 18 weeks - End June 5, 1990

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-346

UNIT NAME Davis-Besse #1

DATE October 11, 1989COMPLETED BY Bilal Sarsour

TELEPHONE (419) 321-7384

REPORT MONTH September 1989

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
7	89-09-22	S	33.3	B	5	N/A	N/A	N/A	Power reduction to 24% to add oil to RCP 2-2 lower motor bearing, and to secure steam leak by backseating valves SP9B1 and SP9A1. See Operational Summary for further details.

¹ F: Forced
S: Scheduled

² Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examiner.
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³ Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation from Previous Month
5-Load Reduction
9-Other (Explain)

⁴ Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵ Exhibit I - Same Source
*Report challenges to Power Operated Relief Valves (PORVs) and Pressurizer Code Safety Valves (PCSVs)

OPERATIONAL SUMMARY

September 1989

Reactor power was maintained at approximately 100% full power until 0545 hours on September 3, 1989, when a manual power reduction was initiated to approximately 82% due to low load demand. Reactor power was maintained at this power level until 1100 hours on September 4, 1989, when reactor power was slowly increased to approximately 100% which was achieved at 1342 hours on September 4, 1989, and maintained at this power level until 2400 hours on September 16, 1989, when a manual power reduction to approximately 92% power was initiated to perform Control Rod Drive (CRD) exercise testing.

After the completion of the CRD exercise testing reactor power was slowly increased to approximately 100% at 1100 hours on September 17, 1989.

Reactor power was maintained at approximately 100% full power until 1600 hours on September 22, 1989, when a manual power reduction was initiated. Reactor power was reduced to approximately 85% to perform turbine control valve testing.

After completion of turbine valve testing, reactor power was further reduced to approximately 24% power which was achieved at 0542 hours on September 23, 1989. During the power reduction, the following are the more significant activities performed:

- 1) Added oil to Reactor Coolant Pump 2-2 lower motor bearing.
- 2) Secured steam leak by backseating valves SP9B1 and SP9A1.
- 3) Repaired the main feedwater control valve (SP6B) positioner air leak.
- 4) Replaced reactor coolant loop 2 hot leg flow transmitter (FT RC1A2).

At 0115 hours on September 24, 1989, a reactor power escalation began. Reactor power was slowly increased to approximately 100% full power which was achieved at 0010 hours on September 25, 1989, and maintained at this power level for the rest of the month.

REFUELING INFORMATION

Date: September 1989

1. Name of facility: Davis-Besse Unit 1
2. Scheduled date for next refueling outage? February 1990
3. Scheduled date for restart from current refueling: N/A
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what in general will these be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

Ans: A license amendment request to remove cycle-specific values from Section 3 of the Technical Specifications was submitted to the NRC on June 16, 1989, based on Generic Letter 88-16. Assuming approval of this submittal by November 1989, no Cycle 7 Technical Specification changes are expected for Section 3 of the Technical Specifications. Cycle-specific changes to Section 2 of Technical Specifications will likely be made. This depends on the date of approval of a document on criteria and methodology, which is presently under development by the B&W Owners Group.

5. Scheduled date(s) for submitting proposed licensing action and supporting information: For Section 2, January, 1990.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
 - a. Sixty Batch Reload 3.38% enriched.
 - b. New fuel design Mark B8A (Reconstitutible, removable upper end fitting, Zircaloy grid spacer, debris resistant lower end cap, lower prepressurization, and annealed guide tubes.
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool, and (c) the new fuel storage areas.

(a) 177 (b) 268 (c) 0
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

Present: 735 Increased size by: approximately 900 by 1993 is planned
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

Date: 1996 - assuming ability to unload the entire core into the spent fuel pool is maintained