



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION
CITY OF ALACHUA
CITY OF BUSHNELL
CITY OF GAINESVILLE
CITY OF KISSIMMEE
CITY OF LEESBURG
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH
CITY OF OCALA
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO
SEBRING UTILITIES COMMISSION
SEMINOLE ELECTRIC COOPERATIVE, INC.
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 124
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power Corporation, et al. (the licensees) dated July 26, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

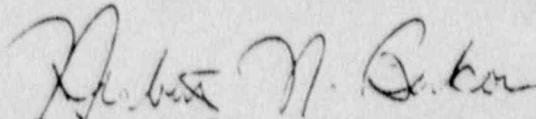
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 124, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - 1/11
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 17, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 124

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove

3/4 3-37

3/4 3-38

3/4 3-39

Insert

3/4 3-37

3/4 3-38

3/4 3-39

INSTRUMENTATION

POST-ACCIDENT INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.3.6 The post-accident monitoring instrumentation channels shown in Table 3.3-10 shall be OPERABLE with reaccouts on all channels in the control room. Recorders on instruments 1 through 10 shall be OPERABLE.

APPLICABILITY: MODES 2, and 3.

ACTION:

- a. With the number of OPERABLE post-accident monitoring channels less than required by Table 3.3-10 (except for Reactor Vessel Level Instrumentation, items 21 and 22), either restore the inoperable channel to OPERABLE status within 30 days, or be in HOT SHUTDOWN within the next 12 hours,
- b. With the number of OPERABLE channels of Reactor Vessel Hot Leg Level or Reactor Vessel Head Level 1 less than required by Table 3.3-10, either restore the inoperable channel to OPERABLE status within 7 days or submit a report to the Commission within the next 30 days outlining the cause of the inoperability and the plans and schedule for restoring the channel to OPERABLE status.
- c. With the number of OPERABLE channels of Reactor Vessel Hot Leg Level or Reactor Vessel Head Level 2 less than required by Table 3.3-10, either restore at least 1 channel to OPERABLE status within 7 days, or be in HOT SHUTDOWN within the next 12 hours,
- d. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.6 Each post-accident monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3-7.

TABLE 3.3-10

POST-ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MEASUREMENT RANGE</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Power Range Nuclear Flux	0-125%	2
2. Reactor Building Pressure	0-70 psia 0-280 psig	2 2
3. Source Range Nuclear Flux	10^{-1} to 10^6 cps	2
4. Reactor Coolant Outlet Temperature	520°F - 620°F	2 per loop
5. Reactor Coolant Total Flow	0-160 x 10^6 lb./hr.	1
6. RC Loop Pressure	0-2500 psig 0-600 psig 1700-2500 psig	2 1 2
7. Pressurizer level	0-320 inches	2
8. Steam Generator Outlet Pressure	0-1200 psig	2/steam generator
9. Steam Generator Operating Range Level	0-100%	2/steam generator
10. Borated Water Storage Tank Level	0-50 feet	2
11. Startup Feedwater Flow	0-1.5x 10^6 lb./hr.	2
12. Reactor Coolant System Subcooling Margin Monitor	-658°F to +658°F	1
13. PORV Position Indicator (Primary Detector)	N/A	1
14. PORV Position Indicator (Backup Detector)	N/A	N/A
15. PORV Block Valve Position Indicator	N/A	N/A
16. Safety Valve Position Indicator (Primary Detector)	N/A	1/valve
17. Safety Valve Position Indicator (Backup Detector)	N/A	N/A
18. Emergency Feedwater Flow	0-850 gpm	2/steam generator
19. Reactor Building Flood Level	0-10 feet	2
20. Core Exit Thermocouples	0-2500°F	2/quadrant
21. Reactor Vessel Hot Leg Level	0-100%	2
22. Reactor Vessel Head Level	0-100%	2

TABLE 4.3-7
POST-ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Power Range Nuclear Flux	M	Q*
2. Reactor Building Pressure	M	R
3. Source Range Nuclear Flux	M	P*
4. Reactor Coolant Outlet Temperature	M	R
5. Reactor Coolant Total Flow Rate	M	R
6. RC Loop Pressure	M	P
7. Pressurizer Level	M	R
8. Steam Generator Outlet Pressure	M	R
9. Steam Generator Level	M	R
10. Borated Water Storage Tank Level	M	R
11. Startup Feedwater Flow Rate	M	R
12. Reactor Coolant System Subcooling Margin Monitor	M	R
13. PORV Position Indicator (Primary Detector)	M	R
14. PORV Position Indicator (Backup Detector)	M	R
15. PORV Block Valve Position Indicator	M	R
16. Safety Valve Position Indicator (Primary Detector)	M	R
17. Safety Valve Position Indicator (Backup Detector)	M	R
18. Emergency Feedwater Flow	M	R
19. Reactor Building Flood Level	M	R
20. Core Exit Thermocouples	M	R
21. Reactor Vessel Hot Leg Level	NA	R
22. Reactor Vessel Head Level	NA	R

*Neutron detectors may be excluded from CHANNEL CALIBRATION

CRYSTAL RIVER UNIT 3

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Amendment No. 28, 67, 72,
77, 78, 79, 84