

#### **BOSTON EDISON**

Pilgrim Nuclear Power Station Rocky Hill Road Plymouth, Massachusetts 02360

K. L. Highfill Station Director

> October 12, 1989 BECo Ltr. #89-151

U.S. Nuclear Regulatory Commission Attention: Document Control Desk Washington, DC 20555

> License No. DPR-35 Docket No. 50-293

Subject: September 1989 Monthly Report

Dear Sir:

In accordance with PNPS Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report please contact me directly.

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WJM:bal

Attachment

cc: Regional Administrator, Region 1 U.S. Nuclear Regulatory Commission 475 Allendale Rd. King of Prussia, PA 19406

Senior Resident Inspector

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## AVERAGE DAILY UNIT POWER LEVEL

MONTH September 1989

DOCKET NO.	50-293
UNIT	Pilgrim 1
DATE	October 12, 1989
COMPLETED	BY W. Munro
TELEPHONE	(508) 747-8474

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	433
2	0	18	481
3	0	19	418
4	0	20	482
5	0	21	481
6	0	22	447
7	61	23	486
8	367	24	486
9	443	25	485
10	401	26	479
11	485	27	482
12	464	28	485
13	457	29	485
14	483	30	485
15	352	31	N/A
16	432		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

## OPERATING DATA REPORT

DOCKET NO.	50-293
DATE	- Ost of man 12, 1080
COMPLETED I	BY W. Munro'
TELEPHONE	(508) 747-8474

# OPERATING STATUS

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	<b>FN</b> 1			
				а.
- 51	-	-	-	-

2.34.5.6.7.8.	Reporting Period September 1989 Licensed Thermal Power (MKt) Nameplate Rating (Gross MWe) Drsign Electrical Rating (Net MWe) Maximum Dependable Capacity (Gross MWe) Maximum Dependable Capacity (Net MWe) If Changes Occur in Capacity Ratings (It Report, Give Reasons: None Power Level To Which Restricted, If Any	1998 678 655 690 670 ems Number 3	Through 7) S	ince Last
io. Act	Reasons For Restrictions, If Any Regulat ion Letter 86-10. Power Ascension Phase.	ory Restricti	on. Confirma	itory
		This Month	Yr-to-Date	Cumulative
111. 12. 13. 14. 15. 16. 17. 18. 19. 20. 21. 22. 23. 24. Mai	Hours In Reporting Period Number Of Hours Reactor Was Critical Reactor Reserve Shutdown Hours Hours Generator On-Line Unit Reserve Shutdown Hours Gross Thermal Energy Generated(MWH) Gross Electrical Energy Generated(MWH) Net Electrical Energy Generated (MWH) Unit Service Factor Unit Availability Factor Unit Capacity Factor (Using MDC Net) Unit Capacity Factor (Using DER Net) Unit Fo:ced Outage Rate Shutdowns Scheduled Over Next 6 Months ( ntenance outage, October 14, 1989 for a d	720.0 577.7 0.0 565.7 0.0 793272.0 264730.0 253466.0 78.6 78.6 52.5 53.7 21.4 Type, Date, a furation of 18	6551.0 4083.0 0.0 3437.9 0.0 2899680.0 904190.0 854777.0 52.5 52.5 19.5 19.9 29.6 ind Duration days	147359.0 83874.4 0.0 80667.2 0.0 137899560.0 46348794.0 44530206.0 54.7 54.7 54.7 45.1 46.1 13.0 of Each):
25.26.	If Shut Down At End Of Report Period, Es Units In Test Status (Prior to Commercia INITIAL CRITICALITY INITIAL ELECTRICITY COMMERCIAL OPERATION	timated Date 1 Operation): F	of Startup · N/A orecast Ad	chieved

\* Includes adjustment from August 1989 Report.

#### BOSTON EDISON COMPANY PILGRIM NUCLEAR POWER STATION DOCKET NO. 50-293

### Operational Summary for September 1989

The station started the month in cold shutdown and remained in an outage condition until 2216 hours on September 6, 1989 when the unit was made critical. At 1015 hours on September 7, 1989 the turbine generator was synchronized to the grid, and by September 8, 1989 the reactor was operating at 75 percent rated thermal power, in accordance with the approved Power Ascension Plan. However, main condenser vacuum problems continue to impact unit availability due to backwashing and associated power reductions. The unit remained on line for the remainder of the reporting period.

> Safety Relief Valve Challenges Month of September 1989

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during the month.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure, auto signal (ADS) or control switch (manual). Ref. BECo ltr. #81-01 dated 01/05/81.

### REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that, each number corresponds to equivalent notation utilized in the request.

- The name of this facility is Filgrim Nuclear Power Station, Docket Number 50-293.
- 2. Scheduled date for next Refueling Shutdown: Second Quarter 1991
- 3. Scheduled date for restart following refueling: Second Quarter 1991
- Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.
- 5. See #6.

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- The new fuel loaded during the 1986/87 refueling outage was of the same design as loaded in the previous outage, and consisted of 192 assemblies.
- 7. (a) There are 580 fuel assemblies in the core.
  - (b) There are 1320 fuel assemblies in the spent fuel pool.
- (a) The station is presently licensed to store 2320 spent fuel assemblies. The actual usable spent fuel storage capacity is 2320 fuel assemblies.
  - (b) The planned spent fuel storage capacity is 2320 fuel assemblies.
- With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 1000 fuel assemblies.

Month September 1989

## PILGRIM NUCLEAR POWER STATION

MAJOR SAFETY RELATED MAINTENANCE

SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED	
- Residual Heat Removal (RHR) System	Relay 10A-K10A	Unplanned ESF actuation resulting from jumpering of incorrect relay during logic relay testing, (TP 88-78). (F&MR 89-335)	Technician error.	Testing discontinued; circuitry returned to normal. affected valves restored to thier normal position, and diesel generator shutdown.	Procedure TP 88-78 revised to include double verification. (Refer to associated LER).	LER 89-027-30	
High Pressure Coolant Injection (HPCI) System	Ramp Generator Signal Convert- er Module (Woodward Gov- ernor Co. Model #8271-083)	HPCI system turbine trip on overspeed during routine surveillance test (Proc. 8.5.4.1) (F&MR 89-339)	Faulty Ramp Generator Signal Converter resul- ting in a mech- anical over- speed trip.	The Ramp Generator Signal Converter (RGSC) (Woodward Governor Co. Model #8271-083) was re- placed with a new RGSC module (Model #9903-015). HPCI turbine speed control system was calibrated per Proc. 8.F.23.1	Refer to associated LER.	LER 89-028-00	

## UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-293	
NAME Pilgrim 1	
DATE October 12, 1989	
COMPLETED BY W. Munro	
TELEPHONE (508) 747-8474	

## REPORT MUNTH September 1989

NO.	DATE	TYPE1	DURATION (HOURS)	REASON <sup>2</sup>	METHOD OF SHUTTING DOWN REACTOR <sup>3</sup>	LICENSE EVENT REPORT #	SYSTEM CODE <sup>4</sup>	COMPONENT CODE5	CAUSE & CORRECTIVE ACTION TO PREVENTIVE RECURRENCE
12	09/01/89 (continued	, F	154.3	A	3	89-026-00	EL	60	Failed potential trans- former combined with main generator voltage balance relay wiring error. Wiring error corrected, potential tranformer replaced. (Ref. LER89-026-00)
13	09/15/89	N/A	0.0	H	5	N/A	SG	N/A	Power reduction to back- wash the condenser.
1		2		2	3	485			
F-Fr S-Si	orced A-Eq ched B-Ma C-Re D-Re E-Op &	uip Fai int or fueling gulator License	lure Test y Restricti Training Examinatio	F-Admin G-Oper Error H-Other on	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit Instruct Preparat Data Ent Licensee (LER) Fi	F & H tions for tion of try Sheet e Event Report ile (NUREG-10	rt )22)	