



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 139 TO FACILITY OPERATING LICENSE NO. DPR-59

POWER AUTHORITY OF THE STATE OF NEW YORK

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

INTRODUCTION

By letter dated May 19, 1989, the Power Authority of the State of New York (PASNY or the licensee), requested changes to the Technical Specifications (TS) for the James A. FitzPatrick Nuclear Power Plant. The changes would remove a misleading reference to a non-existent continuous leak rate monitoring system, change the title of Specification 4.7.A.3 "Continuous Leak Rate Monitor" to "Continuous Leak Rate Monitoring," and delete the sentence "The monitoring system may be taken out of service for maintenance, but shall be returned to service as soon as possible." For clarity, the remaining sentence in the specification would be moved to the preceding page and placed below the title.

DESCRIPTION

The design of the FitzPatrick Nuclear Power Plant primary containment does not include a dedicated leakage monitoring system; i.e., a system which is designed to determine leakage across the boundary of the primary containment. Leakage monitoring is performed by periodically determining the makeup requirements of the Containment Air Dilution (CAD) Inerting System. As described in the Final Safety Analysis Report (FSAR) Section 5.2.3.13, the method consists of measuring the gas flow needed to maintain the primary containment pressure greater than 1.7 psi above the suppression chamber pressure (hence 1.7 psi above atmospheric pressure) as the pressure slowly decays due to minor leakage. This method, as implemented in Operations Department Procedure Number OP-40D, consists of daily logging the CAD nitrogen flow integrator reading, the drywell pressure, the drywell temperature and the drywell humidity by the Control Room operator. The readings are checked by the shift supervisor. If the readings are not within the expected range for the existing plant conditions, an immediate investigation is started. In addition, the differential pressure between the drywell and suppression chamber is logged and checked for possible leakage into the suppression chamber.

This method of leak rate monitoring is not associated with the Reactor Coolant System Leakage Detection System, which measures leakage from the Reactor Coolant System within the drywell and is unaffected by this proposed change.

Based on this analysis it has been determined that the proposed TS change does not alter the conclusions or analysis of either the FitzPatrick FSAR or the Safety Evaluation Report. The change is, therefore, acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in administrative procedures or requirements. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: October 2, 1989

PRINCIPAL CONTRIBUTOR:

D. LaBarge