



UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, D. C. 20555

July 2, 1984

Handwritten notes:
S. L. ...
cc ...

Handwritten initials: E. J. ...

Handwritten signature: Baker

MEMORANDUM FOR: William J. Dircks
Executive Director for Operations
ATTN: T. Rehm
FROM: R. F. Fruey, Executive Director, ACRS
SUBJECT: 290TH ACRS MEETING FOLLOW-UP ITEMS

Based on discussions regarding methods for improved implementation and followup of ACRS recommendations, the Committee agreed that a summary of Actions, Agreements, Assignments, and Requests made during each full Committee meeting will be sent to your office following each meeting.

Attached per this agreement is a list of the requests made at the 290th ACRS Meeting, June 14-16, 1984.

Those items in the list "Actions, Agreements, Assignments, and Requests" dated June 21, 1984 that do not deal with requests made of the NRC Staff or that are not pertinent to NRC Staff activities have not been included in this followup list.

Attachments: As stated

- cc w/attachments:
- C. J. Heltenes, AEOD
- H. R. Denton, NRR
- R. B. Minogue, RES
- R. C. DeYoung, II
- ✓ J. G. Davis, NMSS
- E. G. Case, NRR
- ACRS Members

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ACTIONS, RECOMMENDATIONS, AND REQUESTS
290TH ACRS MEETING, JUNE 14-16, 1984

1. ACRS Report on Diablo Canyon Nuclear Power Plant

The Committee prepared a report to the Commissioners of its review of the proposed license condition that would require Pacific Gas and Electric Company (PG&E) to do a seismic study to reassess the Diablo Canyon design bases and address Commissioner Bernthal's invitation for ACRS comments on the appropriateness of the utility taking the lead in this project. The Committee also reviewed a recent technical paper by J. K. Crouch, S. Backman, and J. Shay regarding the nature of the Hosgri Fault (thrust vs strike-slip), and testimony given before the Commission on this paper. The ACRS requested the opportunity to review and comment on the PG&E program plan and schedule for implementation required by January 30, 1985, and to meet with the NRC Staff as appropriate to discuss their review and evaluation of the PG&E work. A copy of the report mentioned above is attached.

2. ACRS Comments on the NRC Safety Research Program Budget for Fiscal Years 1986 and 1987

The Committee prepared the enclosed report to the Commissioners transmitting its comments on the Program Support Budget of the Office of Nuclear Regulatory Research for Fiscal Years 1986 and 1987.

3. ACRS Report on Proposed Rules Regarding the Use of High Enrichment Uranium in Non-Power Reactors

The Committee prepared a report to the Commissioners of its review of a proposed rule for limiting the use of high enriched uranium in research and test reactors and a plan to increase security measures at non-power reactors that use high-enrichment fuel. M. W. Carbon did not participate in this review. A copy of the report is appended.

4. ACRS Action on Regulatory Guides and Regulations

The Committee prepared the attached letter to the EDO in which it concurred in the regulatory positions of Regulatory Guide 1.35, Revision 3, "Inservice Inspection of UngROUTED Tendons in Prestressed Concrete Containments" and Regulatory Guide 1.35.1, "Determining Prestressing Forces for Inspection of Prestressed Concrete Containments." The ACRS also indicated in that letter that it had no objection to issuing for public comment the proposed General Revision to Appendix J to 10 CFR Part 50, "Leak Tests for Primary and Secondary Containments of Light-Water-Cooled Nuclear Power Plants;" subsequent to the public comment period, the ACRS will review this rule together with the public comments and the NRC Staff's responses to them.

ACTIONS, RECOMMENDATIONS, AND REQUESTS
290TH ACRS MEETING, JUNE 14-16, 1984

5. NRC Program for Management of Plant-Specific Backfitting of Operating Power Reactors

The Committee prepared a memorandum to the EDO requesting an opportunity to review the experience gained in the implementation of the interim procedures which are now in effect for controlling plant-specific backfit proposals for operating reactors, as well as any revisions proposed as a result of comments received during the public comment period just concluded. A copy of the above mentioned memorandum is enclosed.

6. Steam Generator Overfill

The Committee prepared the attached memorandum from the ACRS Executive Director to the EDO transmitting several questions regarding the NRC Staff's approach to addressing the potential for steam generator overfill in PWRs in various generic tasks. The ACRS indicated that it wishes the opportunity to review any results that may be available from efforts on these generic tasks.

7. AEOD Report on Operating Experience Related to Moisture Intrusion in Electrical Equipment at Commercial Power Reactors, July 11, 1984

The Committee discussed a memorandum from W. J. Dircks to the Commissioners, "AEOD Report on Operating Experience Related to Moisture Intrusion in Electrical Equipment at Commercial Power Reactors," dated July 11, 1984. Action on this matter was deferred pending the results of a meeting of the Subcommittee on Qualification Program for Safety-Related Equipment on August 8, 1984.

8. Security of Reactors

The issue of sabotage at research and test reactors was discussed briefly. The Committee members agreed that the issue of sabotage should be reexamined with particular emphasis regarding the status of commercial nuclear power plants. The Subcommittee on Safeguards and Security was asked to schedule a meeting at an appropriate time to discuss this matter. Members of the ACRS Staff will contact members of your staff to set up this meeting.

9. Diablo Canyon Nuclear Power Plant

The NRC Staff was asked to provide a copy of a draft PRA procedures guide for a seismic PRA which L. Reiter, Geosciences Branch, indicated would be available in three months.

During this review it was noted that there is calcium silicate insulation in the Diablo Canyon containment that is not covered by a stainless steel jacket but encapsulated with wire mesh. The Staff was asked to provide an evaluation of the potential for this insulation material breaking loose during a LOCA and resulting in damage to the ECCS recirculation pumps.

ACTIONS, RECOMMENDATIONS, AND REQUESTS
290TH ACRS MEETING, JUNE 14-16, 1984

10. Backfitting of Nuclear Power Plant Changes

The members agreed that it would be appropriate for the ACRS to review the experience gained in the use of NRC Manual Chapter 0514, NRC Program for Management of Plant Specific Backfitting of Operating Power Reactors and any proposed changes in the chapter resulting from the public comment period which has recently ended. The Committee requested an opportunity to conduct this review prior to Commission action regarding this matter.

DOE SOURCE SELECTION JUSTIFICATION

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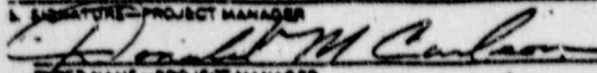
2. PAY TITLE Consequences of Sabotage at
Nonpower Reactors.

3. SELECTED SOURCE
Los Alamos National Laboratory

4. ORGANIZATION (Office, Division and Branch)

NMSS, Division of Safeguards, Fuel Facility Safeguards Licensing Branch

5. SIGNATURE - PROJECT MANAGER



TYPED NAME - PROJECT MANAGER

Donald M. Carlson

DATE

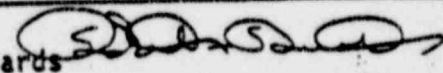
7/6/84

6. BASIS FOR SELECTION (Describe the factors used to justify the basis for selection, see Bulletin 1401-1)

In 1979, Los Alamos National Laboratory (then Los Alamos Scientific Laboratory) conducted a study on the consequences of sabotage at nonpower reactors. NUREG/CR-0843 was published reporting the results of the study. Los Alamos National Laboratory (LANL) personnel have extensive experience in determining releases of radioactive fission product materials as well as the effects of explosives and incendiaries on reactor components. The further evaluation of the potential consequences of sabotage at nonpower reactors represents a logical extension of LANL's prior work in this area.

7. APPROVAL (Signature)

Robert F. Burnett, Director, Division of Safeguards



DATE

8/27/84

8. APPROVED (Signature of Office Director or Designee)

R. S. Brown, Jr., Assistant to the Director and Chief, Program Support Branch

DATE

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