U.S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) 

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AEPORT L 6 0 5 0 0 0 3 6 6 0 0 5 2 3 8 0 8 0 6 1 0 8 0 9 0 9 0 9 0 9 0 9 0 9 1 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) 7] During a power increase, at 1750 MWT, P.S.W. flow was throttled to re-31 duce dillution flow. The Reactor Building Chiller tripped reducing the I T cooling capacity to the D.W. coolers. The average D.W. temp. exceeded ,145 degrees F. violating Tech. Specs. 3.6.1.7. There were no effects 5 a jupon public health and safety due to this event. 7 8 1 SYSTEM CAUSE CAUSE COMP VALVE COMPONENT CODE SUBCODE AIAI A (12 A (13) | H | T | E | X | C | H |(14) 9 G | (15 Z (16) OCCURRENCE REVISION REFORT EVENT YEAR REPORT NO CODE TYPE LER/RO NO. 03 REPORT 0 8 6 0 NUMBER FUTURE SHUTDOWN COMPONENT ATTACHMENT SUBMITTED NPRD-4 PRIME COMP EFFECT ON PLANT HOURS (22) FORM SUB Y 23 7 (21) 0 0 0 0 N (24) A (25 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) O Operators throttled service water flow through the plant causing Rx. Bldg. chillers to trip on high head pressure. This loss of cooling from chilly 2] Water system to D.W. cooling units caused D.W. temp. to increase to 145.1 3162.°F. P.S.W. flow was increased, Rx. Bldg. chiller restarted and reduced temp. to less than 145° F. within 15 minutes. 80 8 METHOD OF FACILITY OTHER STATUS (30) DISCOVERY DESCRIPTION (32) % POWER 1 (29) A (31) Operator Observation 0 7 NA 80 ACTIVITY CONTENT AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36) RELEASED\_OF RELEASE NA Z (33) Z (34) NA 80 PERSONNEL EXPOSURES DESCRIPTION (39) NUMBER 10 0 0 37 NA PERSONNEL INJURIES DESCRIPTION (41) 0 0 0 0 NA 80 LOSS OF OR DAMAGE TO FACILITY (43) DESCRIPTION 0 2 42 NA PUBLICITY 8006200 444 NRC USE ONLY DESCRIPTION (45) NA 1(14 68 69 NAME OF POFPAGED S. X. Baxley, Supt. Operations PHONE 912-367-7781

Georgia Power Company Plant E. I. Hatch Baxley, Georgia

## NARRATIVE REPORT

## Reportable Occurrence Report No. 50-366/1980-086

During a power increase, the P.S.W. plant outlet valves 2N71-FD12 (make-up to the circulating water flume) and 2N71-FD13 (direct discharge to the river) were being realigned to make-up to the flume after a previous alignment for Radwaste dillution flow discharge. During the realignment to flume make-up, the two outlet valves were inadvertantly throttled partially closed simultaneously reducing the total P.S.W. flow. The Reactor Building chiller compressor head pressure increased to the point of a trip due to the lack of cooling. Upon investigation of chiller operation, it was determined that there wasn't enough P.S.W. flow to the chiller. The flume make-up isolation was opened fully and the chiller was restarted and D.W. temperature was returned to < 145° F. within 15 minutes.

Operators have been instructed to insure that one of the outlet valves are full open before closing or adjusting the opposite outlet valve.

This event is not a reoccurring problem and is not applicable to the other unit.

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