



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION III
799 ROOSEVELT ROAD
GLEN ELLYN, ILLINOIS 60137

TIC

APR 18 1980

Docket No. 50-155

Consumers Power Company
ATTN: Mr. R. B. DeWitt
Vice President
Nuclear Operations
212 West Michigan Avenue
Jackson, MI 49201

Gentlemen:

The enclosed Circular No. 80-08 is forwarded to you for information. No written response is required. Should you have any questions related to your understanding of the recommendations on this matter, please contact this office.

Sincerely,

James G. Keppler
James G. Keppler
Director

Enclosure: IE Circular
No. 80-08

cc w/encl:
Mr. D. P. Hoffman, Nuclear
Licensing Administrator
Mr. C. J. Hartman, Plant
Superintendent
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
WASHINGTON, D.C. 20555

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DUPLICATE

April 18, 1980

IE Circular No. 80-08

BWR TECHNICAL SPECIFICATION INCONSISTENCY - RPS RESPONSE TIME

Description of Circumstances:

On March 10, 1980, Vermont Yankee (VY) notified the NRC that the reactor protection system (RPS) response time of 100 milliseconds (from sensor contact opening to and including scram solenoid relay contact opening) specified in the Technical Specification is inconsistent with the value of 50 milliseconds used in the safety analysis. VY verified that actual response times have been below 50 milliseconds since initial plant operation and initiated a Technical Specification change request and plant procedure changes. VY reported that a letter from General Electric Company (GE) informed them of the inconsistency and that the problem may be generic.

Our followup indicates that GE BWR facilities which have Technical Specifications similar to that discussed above may be affected. Based on RPS design and test results, actual RPS response times are expected to be less than 50 milliseconds.

For GE BWR's, we request that you take the following corrective action promptly after receipt of this Circular: (1) verify that the actual RPS response time in the most recent test is less than the value specified in the safety analysis, (2) observe the RPS response time value specified in the safety analysis until a Technical Specification change (if necessary) is approved, and (3) take appropriate actions to make Technical Specification on RPS response time consistent with the RPS response time used in the safety analysis. If a value less than that currently in the Technical Specifications is proposed, the licensee will be expected to provide the basis for that value, including the validity of tests and methods.

No written response to this Circular is required, however, IE will inspect licensee corrective action. If you require additional information regarding this matter, contact the Director of the appropriate NRC Regional Office.

IE Circular No. 80-08
April 18, 1980

Enclosure

RECENTLY ISSUED
IE CIRCULARS

Circular No.	Subject	Date of Issue	Issued to
80-07	Problems with HPCI Turbine Oil System	4/3/80	All holders of a power reactor OL or CP
80-06	Control and Accountability Systems for Implant Therapy Sources	4/14/80	Medical licensees in Categories G and G1
80-05	Emergency Diesel-Generator Lubricating Oil Addition and Onsite Supply	4/1/80	All holders of a power reactor OL or CP
80-04	Securing of Threaded Locking Devices on Safety-Related Equipment	3/14/80	All holders of a power reactor OL or CP
80-03	Protection from Toxic Gas Hazards	3/6/80	All holders of a power reactor OL
80-02	Nuclear Power Plant Staff Work Hours	2/1/80	All holders of Reactor OLs, including research and test reactors, and CPs
80-01	Service Advice for GE Induction Disc Relays	1/17/80	All licensees of nuclear power reactor operating facilities and holders of nuclear power reactor CPs
79-25	Shock Arrestor Strut Assembly Interference	12/20/79	All licensees and holders of power reactor CPs
79-24	Proper Installation and Calibration of Core Spray Pipe Break Detection Equipment on BWRs.	11/26/79	All Holders of a Power Reactor OL or CP
79-23	Motor Starters and Contactors Failed to Operate	11/26/79	All Power Reactor Operating Facilities and Holders of Reactor CPs