



72

UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION I
631 PARK AVENUE
KING OF PRUSSIA, PENNSYLVANIA 19406

April 18, 1980

Docket No. 50-333

Power Authority of the State of New York
James A. FitzPatrick Nuclear Power Plant
ATTN: Mr. R. J. Pasternak
Resident Manager
P. O. Box 41
Lycoming, New York 13093

Gentlemen:

The enclosed IE Circular No. 80-05, "BWR Technical Specification Inconsistency-RPS Response Time," is forwarded to you for information. No written response is required. If you desire additional information regarding this matter, please contact this office.

Sincerely,

James M. Allan
for *Boyce H. Grier*
Director

Enclosures:

1. IE Circular No. 80-08
2. List of Recently Issued IE Circulars

CONTACT: E. Greenman
(215-337-5267)

cc w/encls:

George T. Berry, Executive Director
P. W. Lyon, Manager - Nuclear Operations
A. Klausmann, Director, Quality Assurance
M. C. Cosgrove, Quality Assurance Supervisor
J. F. Davis, Chairman, Safety Review Committee
C. M. Pratt, Assistant General Counsel
G. M. Wilverding, Licensing Supervisor

8006200077

ENCLOSURE 1

UNITED STATES
NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
WASHINGTON, D.C. 20555

SSINS No.: 6830
Accession No.:
80C2280651

DUPLICATE

IE Circular No. 80-08
Date: April 18, 1980
Page 1 of 1

BWR TECHNICAL SPECIFICATION INCONSISTENCY - RPS RESPONSE TIME

Description of Circumstances:

On March 10, 1980, Vermont Yankee (VY) notified the NRC that the reactor protection system (RPS) response time of 100 milliseconds (from sensor contact opening to and including scram solenoid relay contact opening) specified in the Technical Specification is inconsistent with the value of 50 milliseconds used in the safety analysis. VY verified that actual response times have been below 50 milliseconds since initial plant operation and initiated a Technical Specification change request and plant procedure changes. VY reported that a letter from General Electric Company (GE) informed them of the inconsistency and that the problem may be generic.

Our followup indicates that GE BWR facilities which have Technical Specifications similar to that discussed above may be affected. Based on RPS design and test results, actual RPS response times are expected to be less than 50 milliseconds.

For GE BWR's, we request that you take the following corrective action promptly after receipt of this Circular: (1) verify that the actual RPS response time in the most recent test is less than the value specified in the safety analysis, (2) observe the RPS response time value specified in the safety analysis until a Technical Specification change (if necessary) is approved, and (3) take appropriate actions to make Technical Specification on RPS response time consistent with the RPS response time used in the safety analysis. If a value less than that currently in the Technical Specifications is proposed, the licensee will be expected to provide the basis for that value, including the validity of tests and methods.

No written response to this Circular is required, however, IE will inspect licensee corrective action. If you require additional information regarding this matter, contact the Director of the appropriate NRC Regional Office.

ENCLOSURE 2

IE Circular No. 80-08
Date: April 18, 1980
Page 1 of 1

RECENTLY ISSUED IE CIRCULARS

Circular No.	Subject	First Date of Issue	Issued To
79-22	Stroke Times for Power Operated Relief Valves	11/16/79	All Power Reactor Licensees with an Operating License (OL) or Construction Permit (CP)
79-23	Motor Starters and Contactors Failed to Operate	11/26/79	All Power Reactor Licensees with an OL or CP
79-24	Proper Installation and Calibration of Core Spray Pipe Break Detection Equipment on BWRs	11/26/79	All Power Reactor Facilities with an OL or CP
80-01	Service Advice for GE Induction Disc Relays	1/17/80	All Power Reactor Facilities with an OL or CP
80-02	Nuclear Power Plant Staff Work Hours	2/1/80	All Power and Research Reactors with an OL or CP
80-03	Protection from Toxic Gas Hazards	3/6/80	All Power Reactor Facilities with an OL
80-04	Securing Threaded Lacking Devices on Safety-Related Equipment	3/14/80	All Power Reactor Facilities with an OL or CP
80-05	Emergency Diesel Generator Lubricating Oil Addition and Onsite Supply	4/1/80	All Power Reactor Facilities with an OL or CP
80-06	Control and Accountability Systems for Implant Therapy Sources	4/14/80	Medical Licensees in Categories G and G1
80-07	Problems with HPCI Turbine Oil System	4/3/80	All Power Reactor Facilities with an OL or CP