

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

MAY 2 1 1980



Mr. John P. Adams President - General Manager Byron Jackson Pump Division Borg-Warner Corporation 800 West Sixth Street, Suite 950 Los Angeles, California 90017

Dear Mr. Adams:

This is in regard to your request for NRC staff review of Topical Report BJNPQ-101, "Performance and Safety Features of the Byron Jackson Primary Reactor Coolant Pump for Light Water Reactors."

As a result of previous discussions with us concerning the desirability of having a Byron Jackson pump design acceptable for referencing in standard plant designs, you submitted Topical Report BJNPQ-101 for acceptance review by the NRC staff on March 7, 1978. In our letter of March 22, 1978, we informed you that the report was acceptable for NRC staff review. We further stated, "We note that you intend to supplement the report with appendices that will provide information concerning each of the light water reactor nuclear steam supplier systems. Many of the questions that we might ask on the general body of the report would be unnecessary if one or more of the appendices were available. Therefore, we suggest that you submit one of the arpendices as soon as practical."

You have submitted appendices for three reactor coolant pumps as follows:

March 22, 1979 - Appendix BJNPQ-201, C. E. System 80 June 7, 1979 - Apper 'ix BJNPQ-202, G. E. Reactor Size 238 June 7, 1979 - April dix BJNPQ-203, G. E. Reactor Size 251

Recently, you reaffirmed your interest in getting Topical Report BJNPQ-101 approved for referencing and you expressed concern that the staff had not initiated its review. Unfortunately, at the time the appendices to the topical report were submitted, the aftermath of the TMI-2 accident caused severe diversion of staff resources and reorientation of priorities, which is still the case to a large extent. In order to apply the results of lessons learned from the TMI-2 accident to operating reactors, near-term operating licenses and other case work, we do not have sufficient staff resources to devote to the review of topical reports. Further, we are not

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aware at this time of any proposed new standard plant design applications that would warrant a high priority for NRC staff review of topical reports to be reference, in such future applications.

Although we cannot begin a detailed review of your topical report at this time, we did conduct a preliminary examination of the report. Based on this examination it appears that, (1) appropriate initial information has been provided, which in more normal circumstances would have permitted us to begin a detailed review and (2) Topical Report BJNPQ-101, which represents a stateof-the-art pump design, should be acceptable for referencing in standard or custom plant design applications after a detailed NRC staff review of the topical is completed. Until the topical review is completed, applications incorporating a Byron-Jackson pump in the nuclear steam supply system will be evaluated on a case-by-case basis.

We plan to periodically reevaluate our work priorities, including the review of topical reports, and will inform you of any change in the status of topical report reviews. If you have any further questions, please feel free to contact us.

Sincerely,

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Harold R. Denton, Director Office of Nuclear Reactor Regulation