



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

MAY 5 1980

MEMORANDUM FOR: Roger J. Mattson, Director
Division of Safety Technology
Office of Nuclear Reactor Regulation

FROM: Harold R. Denton, Director
Office of Nuclear Reactor Regulation

SUBJECT: LETTER FROM AIF SUBCOMMITTEE ON SAFETY
PARAMETER INTEGRATION

As a matter of NRR policy, activities affecting more than one TMI Action Plan Task (NUREG 0660) and more than one NRR division are to be coordinated by DST. The attached letter from AIF dated April 23, 1980 is an example of such an action.

I am particularly interested in your division coordinating with AIF, other NRR divisions and IE to provide recommendations regarding the following four related items:

- 1) A minimum set of plant status/safety parameters which should be available for display;
- 2) Unique characteristics, if any, of the parameter sets to be displayed at various locations (i.e., Safety Parameter Display Console, Technical Support Center, NRC Operations Center, etc.);
- 3) Should INPO or a reactor vendor be tied in to any of the parameter displays;
- 4) The display capability (i.e., CRT, on-line typewriter, high speed printer, etc.) and the analytic/processing capability required at each location, including the NRC Operations Center.

Please provide your recommendations May 30, 1980.

Harold R. Denton, Director
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc: B. Grimes
D. Eisenhut
S. Hanauer
D. Ross
B. Snyder
R. Vollmer

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ENCLOSURE 1



Atomic Industrial Forum, Inc.
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April 23, 1980

Mr. Harold Denton
Director, Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Division
Washington, D.C. 20555

Dear Harold:

The AIF Policy Committee on Follow-Up to the Three Mile Island Accident has formed a Subcommittee on Safety Parameter Integration, under the chairmanship of Stephen Howell of Consumers Power Company and vice-chairmanship of Warren Owen of Duke Power Company. This Subcommittee will:

- Oversee and coordinate current industry efforts on the content, display, use, and transmission of safety parameter information;
- Articulate the philosophy behind and the goals to be achieved in the activities directly related to safety parameter development and use; and
- Provide a focal point for generic discussions with NRC on the approaches to be used in response to proposed NRC requirements on these issues.

At its first meeting on April 17, 1980, the Subcommittee reviewed the efforts underway by NSAC, the Owners Groups, and IEEE and ANS on the following issues:

- Safety Parameter Display Console;
- Control Room Design Reviews;
- Technical Support Center;
- Emergency Operations Center;
- Nuclear Data Link;

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ENCLOSURE 2



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Watson -
action
Minerals

May 2, 1980

Mr. Harold Denton
 Director, Office of Nuclear Reactor Regulation
 U.S. Nuclear Regulatory Commission
 Washington, D.C. 20555

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Dear Mr. Denton:

In the April 23 letter to you from Byron Lee, Jr., Chairman of the AIF Policy Committee on Follow up to the Three Mile Island Accident, we outlined the activities of our Subcommittee on Safety Parameter Integration. As indicated in the letter, we proposed a meeting with an appropriate NRC management group to discuss:

- (1) A fundamental safety parameter set,
- (2) Functional definitions of NRC Action Plan items related to data display and evaluation, and
- (3) A coherent sequencing schedule for implementing these items.

To facilitate the discussion at the meeting which is presently scheduled for 9:00am May 20, I am enclosing a draft implementation plan/milestone chart for your review prior to the meeting. We have developed reference dates for these steps that are under review and expect to have proposed dates available at the meeting. A principal feature is a two-path program at the outset, which separates the parameter selection process from hardware considerations.

As background information, our Subcommittee on Safety Parameter Integration met in Chicago on April 17 and prepared draft functional definitions and a draft implementation plan which involve the fundamental safety parameters and the different related uses of this information. At our meeting, there was general agreement on the need and value of a simple, integrated system for data acquisition and display needs that could be used in a safety panel or its equivalent; and with supplemental information, in the Technical Support Center (TSC); possibly the Emergency Operations Center (EOC); and the Nuclear Data Link (NDL) if it becomes a requirement. It was agreed that the priority for decision

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