3.3.2.G

Bases: Low Temperature Overpressure Protection

There are 3 means of protecting the RCS from overpressurization by a pressure transient at low temperatures (below 250°F). The first type of protection is ensured by the operation and surveillance of the power operated relief valves with a lift setting of 435 psiq. A single power operated relief valve (PORV) will relieve a pressure transient caused by 1) a mass addition into a solid RCS from a charging pump or 2) a heat input based on a reactor coolant pump being started in an idle RCS and circulating water into a steam generator whose temperature is 50°F greater than the RCS temperature. (1)

The second means of protection is ensured by a PORV being open. It will have the same relieving capabilities as mentioned above.

The third means of protection limits the pressurizer level to 25% and the pressurizer pressure to 100 psig. A pressure transient caused by the inadvertent mass addition from a charging pump running for 10 minutes will be relieved by the large gas volume and low pressure present in the pressurizer as mentioned above. Maintaining the pressurizer level below 25% will also make the hi pressurizer level deviation alarm available to the operator during a mass addition accident.

The repair period of 7 days is based on allowing sufficient time to effect repairs using safe and proper procedures and upon the operability of the redundant porv. The 24 hour time period to reach the restrictive conditions in the pressurizer provides sufficient time to meet these conditions.

The OMS must be treated on a periodic basis consistent with the need for its use. A channel functional test shall be performed prior to enabling the overpressure protection system during cooldown and startup. This test shall be repeated monthly when the PORVs are required to be operable. The channel functional test shall include verification of the backup air supply.

Operability of each low temperature over pressure protection channel requires the control switch to be in the proper position, the pressure point set, the PORV isolation valves open, instrument and solenoid power on, and the PORV backup air supply changed.

The limitations and surveillance requirements on the ECCS equipment provides assurance that a mass addition pressure transfent can be relieved by the operation of a single PORV or the limiting conditions placed on the pressurizer.

The restrictions for startup of a RCP limits the heat input accident to within the relieving capabilities of a single PORV.

 Pressure Mitigating Systems Transient Analysis Results July 1977 Westinghouse Owners Group on RCS Overpressurization.

## 6.6.3.c Special Reports

Reports on the following areas shall be as indicated:

## AREA

## SUBMITTAL DATE

a. Primary Coolant Leakage

Initial report within one year of initial criticality; 5 years after commercial service.

достина выпражения вы выпражения выпражения выпражения в применты в применты в применя в применя в применя в п

b. In-Service Inspection Development

First in-service inspection period after initial full power operation.

c. In-Service Inspection Evaluation

The requirements of Section 11 ASME Boiler Code (IS-620).

d. Loose parts monitoring program

On same date as in-service inspection report.

e. Containment Building Structural Tosting Roport

Within 90 days following completion of each test.

Q

f. Lake Michigan Continuing Lake Study Projects No. 1, 6, 9, 10, 11, 12 \*\*\*

Within 60 days following the end of each 6 month routine operating report period. Each report shall contain tabular and graphical information, analysis and interpretation, of information obtained pursuant to each project.

g. Waukegan Memorial Airport Expansion Plans The expansion plans and status of such plans for the Waukegan Memorial Airport will be reported yearly in the Annual Report including FAA #5010 report within 30 days of operation.

h. Overpressure Protection System Operation

\*\* Zion Nuclear Power Station Environmental Report , Section 2.3.6, Table 9.

SEPCIAL REPORTS

Table 6.6.2

- 3.3 2. Pressurization and System Integrity
  - A. Heatup and Cooldown

The Reactor Coolant System temperature and pressure (with the exception of the pressurizer) shall be limited in accordance with the limit lines shown in Figures 3.3.2-1 and 3.3.2-2 during heatup, cooldown and inservice leak and hydrostatic testing.

- 1. Allowable combinations of pressure and temperature for specified temperature change rates are below and of the right of the limit lines shown. Limit lines for cooldown rates between those presented may be obtained by interpolation.
- 2. Figures 3.3.2-1 and 3.3.2-2
  define limits to assure
  prevention of non-ductile
  failure only. For normal
  operation other inherent
  plant characteristics, e.g.,
  pump heat addition and
  pressurizer heater capacity
  may limit the heatup and
  cooldown rates that can be
  achieve, over certain pressuretemperature ranges.
- The Reactor Coolant System shall be limited to a maximum temperature change of 10°F/hr during inservice

- 4.3 2. Pressurization and System Integrity
  - A. Not Applicable

## .3.2. A3. (Continued)

leak and hydrostatic testing operations above the heatup and cooldown limit curves.

- B. The limit lines shown in Figures 3.3.2-1 and 3.3.2-2 shall be recalculated periodically as required, based on results from the material surveillance program.
- C. The secondary side of the steam generator must be pressurized above 200 psig if the temperature of the vessel is below 70°F.
- D. The pressurizer heatup rate shall not exceed 100°F/hr and the pressurizer cooldown rate not exceed 200°F/hr. The spray shall not be used if the temperature difference between the pressurizer and the spray fluid is greater than 320°F.
- E. Hydrostatic Testing
  - System inservice leak and hydrotests shall be performed in accordance with the requirements of ASME Boiler and Pressure Vessel Code, Section XI, 1974 Edition, up to and including Summer 1975 Addendum.