(7-77)

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	N J O C P 1 (2)	0 0 - 0 0 0 0 0 - 0 0 (3)	4 1 1 1 1 (4)	(5)
		LICENSEE CODE	LICENSE NUMBER	LICENSE TYPE	CAT

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0 1	REPORT SOURCE	L (6)	0 5 0 0 0 2 1 9 (7)	0 5 0 7 8 0 (8)	0 6 0 5 8 0 (9)
			DOCKET NUMBER	EVENT DATE	REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

On May 7, 1980, while performing monthly surveillance testing of the reactor high pressure scram sensors, RE03D was found to be less conservative than specified in the Technical Specifications. The safety significance of this event is considered minimal since RE03D would have actuated, but at a pressure 9 psig higher than the prescribed value of 1066 psig. A redundant sensor in the same system would have actuated at the desired pressure.

SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
I B (11)	X (12)	Z (13)	I N S T R U (14)	S (15)	Z (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 0	—	0 1 8	/	0 3	L	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
E (18)	Z (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	Y (24)	X (25)	B 0 6 9 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

The cause of switch RE03D tripping at 1075 psig is attributed to sensor repeatability. The reactor high pressure scram sensor was reset to trip within the prescribed limits.

FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
H (28)	0 0 0 (29)	NA	B (31)	Monthly Surveillance Test

ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
Z (33)	Z (34)	NA	NA

PERSONNEL EXPOSURES

NUMBER	TYPE	DESCRIPTION (39)
0 0 0 (37)	Z (38)	NA

PERSONNEL INJURIES

NUMBER	DESCRIPTION (41)
0 0 0 (40)	NA

LOSS OF OR DAMAGE TO FACILITY (43)

TYPE	DESCRIPTION
Z (42)	NA

PUBLICITY

ISSUED	DESCRIPTION (45)	NRC USE ONLY
Y (44)	Weekly News Release	

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JCP&L / GPU

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Madison Avenue at Punchbowl Road
Morristown New Jersey 07960
201 539-6111

OYSTER CREEK NUCLEAR GENERATING STATION
Forked River, New Jersey 08731

Licensee Event Report
Reportable Occurrence No. 50-219/80-18/3L

Report Date

June 5, 1980

Occurrence Date

May 7, 1980

Identification of Occurrence

Reactor High Pressure Sensor RE03D trip setting was discovered to be greater than 1060 PSI.

This event is considered to be a reportable occurrence as defined in the Technical Specifications, paragraph 6.9.2.b.1.

Conditions Prior to Occurrence

The plant was shutdown for a refueling/maintenance outage.

The reactor cavity was flooded.
The reactor was subcritical.
The reactor mode switch was locked in the SHUTDOWN position.

Description of Occurrence

On Wednesday, May 7, 1980, at approximately 0117 hours, while performing monthly surveillance testing of the reactor high pressure scram sensors, RE03D was found to be less conservative than that specified in the Technical Specifications. Tests on all reactor high pressure switches revealed the following data:

	Pressure Switch Designation	Desired Set Point	As Found	As Left
System 1	RE03A	1068	1066	1066
	RE03C	1066	1066	1066
System 2	RE03B	1068	1068	1068
	RE03D	1066	1075	1066

Apparent Cause of Occurrence

The cause of switch RE03D tripping at 1075 PSIG is attributed to sensor repeatability.

Analysis of Occurrence

RE03D pressure switch would have actuated but at a pressure 9 PSIG greater than it's prescribed value of 1066 PSIG. The only safety significance of this event was the loss of redundancy in the pressure switch actuation at the prescribed set point.

Corrective Action

Reactor high pressure sensor RE03D was reset to trip within the prescribed limits.

Failure Data

Barksdale Pressure Actuated Switch
Switch #B2T-A12SS

Proof 1800 PSI
Adjustable Range 50-1200 PSI