VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

June 9, 1980

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
Attn: Mr. B. Joe Youngblood, Chief
Licensing Branch No. 1
Division of Licensing
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Serial No. 479
NO/FHT:baw
Docket No. 50-339
License No. NPF-7

Bulletin and Orders Task Force Final Recommendations
North Anna Power Station Unit 2

Dear Mr. Denton:

We have reviewed the NRC Memorandum SECY-80-230, dated May 2, 1980, and presented to the Commissioners in a meeting on May 8, 1980. This memorandum was intended to describe Commission actions required in connection with NUREG-0660, the TMI Action Plan.

Enclosure 1 to this memorandum provided the specific TMI-related requirements and actions for new operating licenses as contained in the Action Plan. The first group of requirements, those for fuel loading and low power testing, included four final recommendations of the Bulletin and Orders (B&O) Task Force listed in Table C.3 of Volume I of the Action Plan: C.3.9, C.3.10, C.3.11 and C.3.12. It was noted that these B&O recommendations were not specifically delineated as fuel loading requirements prior to the review of North Anna Unit 2.

This letter is intended to confirm North Anna Unit 2 compliance with these four B&O Task Force final recommendations (Items C.3.9, C.3.10, C.3.11 and C.3.12).

Item C.3.9

B&O Recommendation: For Westinghouse-designed reactors, modify the pressure (sic) integral derivative controller, if installed on the PORV, to eliminate spurious openings of the PORV.

Response: North Anna Unit 2 is in compliance with this item. The proportional integral differential (PID) controller on the pressurizer PORV has been modified as recommended by Westinghouse. The rate time constant has been set equal to zero, eliminating spurious openings of the PORV.

Item C.3.10

B&O Recommendation: For Westinghouse-designed reactors, if the anticipatory reactor trip upon turbine trip is to be modified to be bypassed at power levels less than 50 percent, rather than below 10 percent as in current designs, demonstrate that the probability of a small-break LOCA resulting from a stuck-open PORV is not significantly changed by this modification.

Response: North Anna Unit 2 is in compliance with this item. The anticipating reactor trip upon turbine trip currently can only be bypassed at power levels below 10 percent.

Item C.3.11

B&O Recommendation: Demonstrate that the PORV installed in the plant has a failure rate that is not significantly less (sic) than the valves for which there is an operating history.

Response: North Anna Unit 2 is in compliance with this item. The installed PORV manufactured by Masoneilan (model no. 3" IA58RGP) is similar in design to the Copes-Vulcan valves used in most Westinghouse-designed plants. Both valves were purchased to a Westinghouse design specification and, based on a review by Westinghouse, would be subject to similar failure mechanisms and rates. In addition, two Masoneilan valves have operated satisfactorily for over two years at North Anna Unit 1. No modification of these valves is required.

Item C.3.12

B&O Recommendation: For Westinghouse-designed reactors, confirm that there is an anticipatory reactor trip on turbine trip.

Response: North Anna Unit 2 is in compliance with this item. There is currently an anticipatory reactor trip upon turbine trip.

If you have any questions or require additional information, please contact this office.

Very truly yours,

B. R. Sylvia
Manager-Nuclear Operations
and Maintenance