



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

MAY 10 1980

Generic Task No. A-12

The Honorable Morris Udall, Chairman
Subcommittee on Energy and the Environment
Committee on Interior and Insular Affairs
United States House of Representatives
Washington, DC 20515

Dear Mr. Chairman:

Enclosed for the information of the Subcommittee on Energy and the Environment is the "For Comment" edition of NUREG-0577, "Potential for Low Fracture Toughness and Lamellar Tearing on PWR Steam Generator and Reactor Coolant Pump Supports". This report provides the staff's resolution of the NRC's Generic Technical Activity A-12, which had been declared an "Unresolved Safety Issue" pursuant to Section 210 of the Energy Reorganization Act of 1974. NUREG-0577 describes the technical issues, the technical studies performed by an NRC contractor, the NRC staff's technical positions on fracture toughness of steam generator and reactor coolant pump support materials based on these studies, and the staff's plans for implementing its technical positions. It also provides recommendations for further generic research regarding lamellar tearing. The Electric Power Research Institute has been asked to fund and manage such research.

We intend a 60 day public comment period for NUREG-0577. Also enclosed for your information is a Federal Register Notice we have issued on this matter.

Sincerely,

A handwritten signature in dark ink, appearing to read "Harold R. Denton", is written over the typed name.

Harold R. Denton, Director
Office of Nuclear Reactor Regulation

Enclosures:

1. NUREG-0577
2. Federal Register Notice

cc: The Honorable Steven D. Symms

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A-12 Subject File

NUCLEAR REGULATORY COMMISSIONNUREG-0577NOTICE OF ISSUANCE AND AVAILABILITYPOTENTIAL FOR LOW FRACTURE TOUGHNESS AND LAMELLAR TEARING
ON PWR STEAM GENERATOR AND REACTOR COOLANT PUMP SUPPORTS

A task force with members from the Nuclear Regulatory Commission (NRC) staff has prepared a report entitled "Potential for Low Fracture Toughness and Lamellar Tearing on PWR Steam Generator and Reactor Coolant Pump Supports" (NUREG-0577), dated August 1979. The report provides the staff's resolution of the NRC's Generic Technical Activity A-12, which was an "Unresolved Safety Issue" pursuant to Section 210 of the Energy Reorganization Act of 1974.

The generic study resulted from questions raised during the licensing review of two pressurized water reactors. The specific concern was the capability of the supports to maintain their structural integrity under accident conditions.

NUREG-0577 describes the technical issues, the technical studies performed by an NRC consultant, the NRC staff's technical positions on fracture toughness of steam generator and reactor coolant pump support materials based on these studies, and the staff's tentative plans for implementing its technical positions. It also provides recommendations for further generic research into the subject of lamellar tearing. The Electric Power Research Institute has been requested to fund and manage such research.

The NRC staff has conducted a review of the generic study, six were reviewed for adequacy of the materials and thus no further action is required.

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