

INTERIM REPORT

Accession No. \_\_\_\_\_  
EGG-CAAP-5140

Contract Program or Project Title: Code Assessment and Applications Program

Subject of this Document: An Inventory of the Two-Phase Critical Flow Experimental Data Base

Type of Document: Preliminary Assessment Report

Author(s): D. G. Hall

Date of Document: April 1980

Responsible NRC Individual and NRC Office or Division: S. Fabric, RES/RSR

This document was prepared primarily for preliminary or internal use. It has not received full review and approval. Since there may be substantive changes, this document should not be considered final.

Prepared for  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

INTERIM REPORT

NRC Research and Technical  
Assistance Report

8006020 552



P.O. BOX 1625, IDAHO FALLS, IDAHO 83415

Mr. R. E. Tiller, Director  
Reactor Operations and Programs Division  
Idaho Operations Office - DOE  
Idaho Falls, ID 83401

TRANSMITTAL OF CODE ASSESSMENT REPORT, "AN INVENTORY OF THE TWO-PHASE  
CRITICAL FLOW EXPERIMENTAL DATA BASE" (CAAP-EGG-5140) (A6047) -  
JAD-90-80

Ref: S. Fabric ltr. to R. E. Tiller, December 28, 1979

Dear Mr. Tiller:

Attached is a report entitled, "An Inventory of the Two-Phase Critical  
Flow Experimental Data Base" by D. G. Hall. The last five sections of  
the report constitute NRC's contribution to the Committee on Safety of  
Nuclear Installations (CSNI) state of the art report (SOAR). Preparation  
of the SOAR contribution was made in response to the referenced letter.  
Copies of the attached report will be forwarded to members of the SOAR  
working group for a peer review to be conducted at CSNI offices in Paris,  
France on May 12 and 13.

Very truly yours,

J. A. Dearien, Manager  
Code Assessment and  
Applications Program

DGH:clj

Attachment:  
As stated

S. Fabric, NRC-RSR  
R. W. Kiehn, EG&G Idaho

bcc: T. R. Charlton  
A. C. Peterson  
L. J. Ybarrondo  
Central File  
~~DGH~~ D. G. Hall file

NRC Research and Technical  
Assistance Report

EGG-CAAP-5140

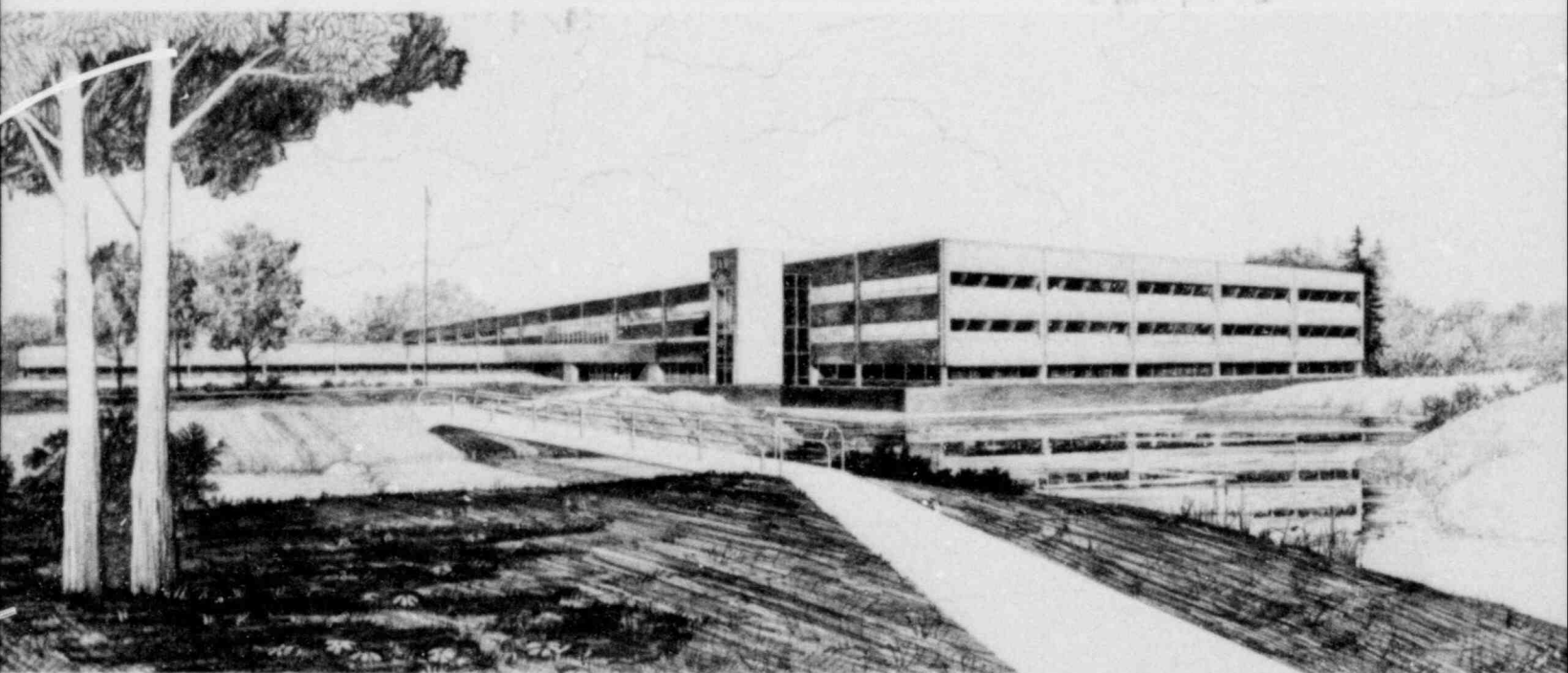
April 1980

AN INVENTORY OF THE TWO-PHASE CRITICAL FLOW  
EXPERIMENTAL DATA BASE

D. G. Hall

**U.S. Department of Energy**

Idaho Operations Office • Idaho National Engineering Laboratory



This is an informal report intended for use as a preliminary or working document

*NRC Research and Technical  
Assistance Report*

Prepared for the  
U.S. Nuclear Regulatory Commission  
Under DOE Contract No. DE-AC07-76ID01570  
NRC FIN No. A6047

 **EG&G** Idaho



FORM EG&G-398  
(Rev. 11-79)

## INTERIM REPORT

Accession No. \_\_\_\_\_

Report No. EGG-CAAP-5140

**Contract Program or Project Title:** Code Assessment and Applications Program

**Subject of this Document:** An Inventory of the Two-Phase Critical Flow Experimental Data Base

**Type of Document:** Preliminary Assessment Report

**Author(s):** D. G. Hall

**Date of Document:** April 1980

**Responsible NRC Individual and NRC Office or Division:** S. Fabric, NRC-RSR

This document was prepared primarily for preliminary or internal use. It has not received full review and approval. Since there may be substantive changes, this document should not be considered final.

EG&G Idaho, Inc.  
Idaho Falls, Idaho 83415

Prepared for the  
U.S. Nuclear Regulatory Commission  
Washington, D.C.  
Under DOE Contract No. DE-AC07-76ID01570  
NRC FIN No. A6047

## INTERIM REPORT

NRC Research and Technical  
Assistance Report ✓

## ABSTRACT

An inventory of currently available experimental critical flow data has been performed. The results of the inventory are displayed in a table which lists key parameters that characterize each experimental program. The distribution of the data base with regard to geometric parameters is presented for three classes of a test section. Recommendations for future testing are made in light of deficiencies that have been identified. Additional recommendations to enhance the utility of the current data base and the results of future experimental programs are made. A bibliography of references documenting experimental critical flow studies is also included.

## ACKNOWLEDGEMENTS

The author wishes to express his thanks to members of the INEL Library reference staff for their thorough and timely acquisition of literature searches and technical documents, to D. L. Terry for assisting with the preparation of the bibliography and report figures, and to J. M. Mosher for typing the various drafts of the report.

## CONTENTS

ABSTRACT.....	ii
ACKNOWLEDGEMENTS.....	iii
SUMMARY.....	v
INTRODUCTION .....	1
DATA BASE .....	3
DISCUSSION .....	12
CONCLUSIONS AND RECOMMENDATIONS .....	17
REFERENCES .....	22
BIBLIOGRAPHY OF EXPERIMENTAL TWO-PHASE CRITICAL FLOW STUDIES.....	26

## FIGURES

1a. Lengths and diameters of constant area test sections.....	13
1b. Length-to-diameter ratio of constant area test sections.....	14
2. Converging - diverging nozzle throat diameters.....	15
3. Orifice diameters.....	16

## TABLE

1. Critical Flow Experimental Data Base.....	4
--	---

## SUMMARY

The phenomenon whereby the flowrate of a two-phase fluid has an upper bound for a given set of stagnation conditions has been studied extensively during the past forty years. It was recently decided by the Committee on the Safety of Nuclear Installations (CSNI) of the Organization for Economic Cooperation and Development (OECD) that the state-of-the-art of modeling this phenomenon known as critical or choked flow will be determined and documented. The task includes an inventory of the experimental critical flow data base. The results of such an inventory are reported herein.

Computerized literature searches were performed on the catalogs of seven technical information services. The resulting bibliography was screened and documents reporting experimental critical flow studies were obtained and reviewed. The data base was documented by producing a large table containing information describing each experimental study. The principal dimensions of three classes of test sections; pipes, nozzles, and orifices have been assembled in graphical form to aid in analyzing the distribution of the geometries that have been studied.

The data base inventory showed that significant amounts of experimental data are available for pipes, nozzles, and orifices and that the range of principal dimensions of the test sections in these three categories is considerable. Deficiencies appear to exist in the areas of critical flow data for nozzles and orifices larger than 30 mm, for standard plumbing components (only two references on this subject were found), and for slits which are representative of cracks in pipe walls and in weldments (one reference on this subject was found). In addition to identifying possible deficiencies in the data base, recommendations are made to increase the utility of the data that is presently available and that which will be produced in the future. These recommendations deal with planning future studies to ensure that they mesh with the data presently available, complete documentation of the stagnation state of the flow, and the inclusion of tabulated data and complete measurement uncertainty information in the reporting document.



The report is concluded with a bibliography of references which appear from their abstracts to contain experimental critical flow data. This bibliography is included since the inventory of the experimental data base presented herein is not considered to be complete because of time constraints in performing the inventory, the unavailability of some references, and the inability of the author to review documents not written in or translated to English.

## INTRODUCTION

The phenomenon of critical or choked flow occurs in a wide range of technologies. Because of its importance, a large number of experimental studies of the phenomenon have been conducted during the last 40 years. The importance of the phenomenon in predicting the results of a nuclear reactor loss-of-coolant accident was primarily responsible for accelerating the rate at which experimental studies were conducted beginning in the 1960s. The results of the experimental studies have been used to develop numerous analytical models of the phenomenon. It is clear to those accustomed to utilizing the existing analytical critical flow models that the problem of predicting the critical flow rate and the thermodynamic state of the choke point is not a closed case for many flow situations of interest. This problem is a result of the large variety of flow geometries of interest and the wide range of fluid conditions over which the phenomenon must be predicted.

In the interest of avoiding duplication of effort, maximizing the utility of past research, and providing direction for future research, it is beneficial to periodically document the level of understanding and the art of predicting a particular physical phenomenon. At its November 1978 meeting the Committee on the Safety of Nuclear Installations (CSNI) of the Organization for Economic Cooperation and Development adopted a proposal by the United States that the Committee should undertake the preparation of state-of-the-art reports (SOAR) on selected topics of interest. At the November 1979 meeting the Committee adopted a list of topics submitted by committee members. It was further decided that two SOARs would be prepared in time for the November 1980 meeting. One of those reports would document the state-of-the-art of critical flow modeling. An outline for the critical flow SOAR was assembled by a group of technical experts in January 1980. It was determined that the document would contain inventories of the critical flow experimental data base and the available analytical models as well as assessments of how well the models predict the critical flow phenomenon.

An inventory of the critical flow experimental data base was performed as the United States Nuclear Regulatory Commission's contribution to the production of the critical flow SOAR. The results of the inventory are documented herein. The principal parameters describing each experimental study that has contributed to the data base are presented in the next section. A discussion of ranges of parameters for which data is available is presented in the third section. The fourth section of the report contains conclusions regarding the availability of experimental critical flow data and recommendations for future experimental work. The report is concluded with a bibliography of references describing experimental critical flow studies.

## DATA BASE INVENTORY

The experimental critical flow data base was inventoried by reviewing as many references documenting experimental critical flow studies as possible. Documents were selected by reviewing the results of computerized literature searches of the catalogs of the following information services for the indicated years:

Nuclear Safety Information Service - 1967-1980  
DOE Energy Data Base - 1974-1980  
Nuclear Science Abstracts - 1967-1976  
Government Reports Announcements (National Technical Information Service) - 1964-1980  
Engineering Index - 1970-1979  
Science Abstracts - 1969-1979

The critical flow data base is summarized in Table 1 which lists parameters that characterize each experimental program. The table does not present a complete inventory of the experimental data base, but does contain many of the experimental data sources that have been referenced in the literature during the past 20 years. The table entries are divided into four groups: pipes, nozzles, orifices, and other geometries. The entries within each group are listed in chronological order from the most recent to the earliest. The author's name appearing on the reference documenting the experiment study has been used to identify each study with an accompanying reference notation referring to an entry in the reference section of the report. Exceptions to this convention have been made in the cases of data generated during the extensive test programs of the Marviken CFT Project and the Semiscale Project. Data from these programs are referred to by the project name. The document publication date is given generally by month and year. This date does not necessarily correspond closely to when the testing was conducted. The general type of test section or flow geometry in which choking occurred is listed followed by the size or range of sizes of the minimum test section cross section.

TABLE 1. CRITICAL FLOW EXPERIMENTAL DATA BASE

SOURCE	DATE	TYPE	SIZE (mm)	FLUID	REGIME	PRESSURE LEVEL (bars)	COMMENTS
Boivin <sup>1</sup>	12-79	pipe (rec)	12-50	H <sub>2</sub> O	sub	p <sub>0</sub> = 20-101	L/D = 38 - 53?; test section lengths not clearly reported
Marviken CFT Project <sup>2</sup>	1978-79	pipe (rec)	200-509	H <sub>2</sub> O	sub & sat (2φ)	p <sub>0</sub> = 40-50	L/D = 0.3 - 3.7
Jeandey and Pinet <sup>3</sup>	6-78	pipe (sec?)	14	H <sub>2</sub> O/N <sub>2</sub>	simulated 2φ	p <sub>p</sub> = 2-6	L/D = 169; stagnation conditions not reported; pressure in upstream portion of the pipe (p <sub>p</sub> ) reported
Ardon and Ackerman <sup>4</sup>	6-78	pipe (sec)	26	H <sub>2</sub> O	sub	p <sub>p</sub> = 1.4	L/D = 39; stagnation conditions not reported; static pressure in upstream portion of the pipe (p <sub>p</sub> ) reported
Reocreux <sup>5</sup>	8-77	pipe (sec)	20	H <sub>2</sub> O	sat (2φ)	p <sub>e</sub> = 1.5-2.0	L/D = 124; stagnation conditions not reported; static pressure at the exit of the constant section (p <sub>e</sub> ) reported
Semiscale Project <sup>6</sup>	6-77	pipe (cec)	18	H <sub>2</sub> O	sub & sat (2φ)	p <sub>0</sub> = 3-103	L/D = 4; system blowdown experiment
Khlestkin, Kanish - chev, and Keller <sup>7</sup>	3-77	pipe (sec)	4	H <sub>2</sub> O	sub & sat	p <sub>0</sub> = 6-228	L/D = 0.5-6.0; flow rates are in nondimensional form
Prisco, Henry, Hutcherson, and Linenhan <sup>8</sup>	3-77	pipe (sec)	20	Freon-11	sub & sat (2φ)	p <sub>0</sub> = 67-115 kPa	L/D = 2.8 - 100.0
Morrison <sup>9</sup>	10-76	pipe (rec)	28	H <sub>2</sub> O	sub & sat (2φ)	p <sub>0</sub> = 56-67	L/D = 4.8

Note: cec = conical entrance contour      sat = saturated liquid state  
 rec = radiused entrance contour      sat (2φ) = saturated two-phase state  
 sec = sharp entrance contour          sub = subcooled state

TABLE 1 Continued

SOURCE	DATE	TYPE	SIZE (mm)	FLUID	REGIME	PRESSURE LEVEL (bars)	COMMENTS
Seynaeve, Giot, and Fritte <sup>10</sup>	8-76	pipe (sec & cec)	12.5, 20	H <sub>2</sub> O	sub	$p_e = 1.4-6.7$	L/D = 17.7 - 124.5; stagnation conditions not reported; static pressure at the exit of the constant area section ( $p_e$ ) reported
Hutcherson <sup>11</sup>	11-75	pipe (cec)	108	H <sub>2</sub> O	sat (2 $\phi$ )	$p_0 = 1-18$	L/D = 3; system blowdown experiment
Sozzi & Sutherland <sup>12</sup>	7-75	pipe (sec & rec)	13	H <sub>2</sub> O	sub & sat (2 $\phi$ )	$p_0 = 30-71$	L/D = 0.4-140
Prisco <sup>13</sup>	2-75	pipe (sec)	8	CCl <sub>3</sub> F	sub?	$p_0 = 67-115$ Pa	L/D = 2.8-12.8
Howard <sup>14</sup>	1-75	pipe (sec & rec)	2-6	Freon-11	sub	$p_0 = 52-165$ kPa	L/D = 25-300
Edwards & Jones <sup>15</sup>	1974	pipe (sec)	32	H <sub>2</sub> O	sat(2 $\phi$ )	$p_0 = 2-54$	L/D = 28; system blowdown experiment
Mal'tsev, Kniestkin, and Keller <sup>16</sup>	6-72	pipe (sec)	3, 3.5	H <sub>2</sub> O	sat	$p_0 = 20-220$	L/D = 0.5-9.0
Klingebiel & Moulton <sup>17</sup>	3-71	pipe (cec)	13	H <sub>2</sub> O	sat (2 $\phi$ )	$p_e = 2-5$	L/D = 44; stagnation conditions not reported; static pressure at exit of constant area section ( $p_e$ ) reported
Henry <sup>18</sup>	9-70	pipe (rec)	8	H <sub>2</sub> O	sub	$p_e = 10-20$	L/D = 115; stagnation conditions not completely reported; static pressure at exit of constant area section ( $p_e$ ) reported
Ailemann et al. <sup>19</sup>	6-70	pipe (sec)	21-173	H <sub>2</sub> O	sub & sat (2 $\phi$ )	$p_0 = 42-165$	L/D = 0.5-4.3; system blowdown experiment

Note: cec = conical entrance contour  
 rec = radiused entrance contour  
 sec = sharp entrance contour  
 sat = saturated liquid state  
 sat (2 $\phi$ ) = saturated two-phase state  
 sub = subcooled state

TABLE 1 Continued

<u>SOURCE</u>	<u>DATE</u>	<u>TYPE</u>	<u>SIZE (mm)</u>	<u>FLUID</u>	<u>REGIME</u>	<u>PRESSURE LEVEL (bars)</u>	<u>COMMENTS</u>
Henry <sup>20</sup>	3-68	pipe (rec)	3, 8	H <sub>2</sub> O	sub	$p_e = 2-10$	L/D = 115, 274; stagnation conditions not fully reported; static pressure at exit of constant area section ( $p_e$ ) reported
Kelly <sup>21</sup>	1-68	pipe (sec?)	2-3	H <sub>2</sub> O	sub & sat (2 $\phi$ )	$p_e = 1-6$	L/D = 90; stagnation conditions not specified; static pressure at exit of constant area section ( $p_e$ ) reported
Uchida & Nariai <sup>22</sup>	8-66	pipe (sec)	4	H <sub>2</sub> O	sub & sat	$p_0 = 0.2-0.8$	L/D = 25-625
Fauske <sup>23</sup>	1965	pipe (sec)	6	H <sub>2</sub> O	sat	$p_0 = 7-124$	L/D = 0-40
Zaloudek <sup>24</sup>	1-64	pipe (cec)	13	H <sub>2</sub> O	sat (2 $\phi$ )	$p_0 = 28-124$	L/D = 20
Zaloudek <sup>25</sup>	5-63	pipe (sec & rec)	6-16	H <sub>2</sub> O	sub	$p_0 = 8-25$	L/D = 1-6
Fauske & Min <sup>26</sup>	1-63	pipe (sec?)	?	Freon-11	sat	$p_0 = 103 \text{ kPa}$	L/D = 2-55
Fauske <sup>27</sup>	10-62	pipe (sec)	3-12	H <sub>2</sub> O	sat (2 $\phi$ )	$p_e = 3-25$	L/D = 228-880; stagnation conditions not reported; static pressure at exit of constant area section ( $p_e$ ) reported
Friedrich & Vetter <sup>28</sup>	1-62	pipe (sec & rec)	4	H <sub>2</sub> O	sub & sat (2 $\phi$ )	$p_0 = 6-30$	L/D = 0.2-15
Friedrich <sup>29</sup>	10-60	pipe (sec & rec)	1.5--	H <sub>2</sub> O	sub & sat (2 $\phi$ )	$p_0 = 2-61$	L/D = 0.2-2.5

Note: cec = conical entrance contour  
 rec = radiused entrance contour  
 sec = sharp entrance contour

sat = saturated liquid state  
 sat (2 $\phi$ ) = saturated two-phase state  
 sub = subcooled state

TABLE 1 Continued

SOURCE	DATE	TYPE	SIZE (mm)	FLUID	REGIME	PRESSURE LEVEL (bars)	COMMENTS
Isbin, Moy, and Da Cruz <sup>30</sup>	9-57	pipe (cec)	10-26	H <sub>2</sub> O	sat	$p_e = 27-296$ kPa	L/D = 23-64 assuming L = 610 mm (2 ft.) stagnation conditions not reported; static pressure at exit of constant area section ( $p_e$ ) reported
Moy <sup>31</sup>	1-55	pipe (cec)	6-24	H <sub>2</sub> O	sat (2 $\phi$ )	$p_e = 27-296$ kPa	L/D = 35-96; stagnation conditions not reported; static pressure at exit of constant area section ( $p_e$ ) reported
Pasqua <sup>32</sup>	5-52	pipe (sec & rec)	1-3	Freon-12	sub & sat	6-9	L/D = 4-24
Burnell <sup>33</sup>	12-47	pipe (sec & rec)	5-38	H <sub>2</sub> O	sat	$p_0 = 1-12$	L/D = 0-656
Danforth <sup>34</sup>	5-41	pipe (rec)	3	H <sub>2</sub> O	sub	$p_0 = 3-7$	L/D = 1
			N O Z Z L E S				
Martinez <sup>35</sup>	12-79	Nozzle	3	Freon-11	sub	$p_0 = 16-22$	
Zimmer et al. <sup>36</sup>	4-79	Nozzle	25	H <sub>2</sub> O	sub	$p_0 = 1-10$	
Semiscale Project <sup>37</sup>	12-78	Nozzle	17	H <sub>2</sub> O	sub & sat	$p_0 = 3-100$	System blowdown experiment
Karasev, Vazinger, and Mingaleeva <sup>38</sup>	6-77	Nozzle	4,19	H <sub>2</sub> O	sat	$p_0 = 20-100$	
Semiscale Project <sup>39</sup>	6-77	Nozzle	25	H <sub>2</sub> O	sub & sat (2 $\phi$ )	$p_0 = 3-90$	System blowdown experiment

Note: cec = conical entrance contour      sat = saturated liquid state  
 rec = radiused entrance contour      sat (2 $\phi$ ) = saturated two-phase state  
 sec = sharp entrance contour      sub = subcooled state



TABLE 1 Continued

<u>SOURCE</u>	<u>DATE</u>	<u>TYPE</u>	<u>SIZE (mm)</u>	<u>FLUID</u>	<u>REGIME</u>	<u>PRESSURE LEVEL (bars)</u>	<u>COMMENTS</u>
Semiscale Project <sup>40</sup>	1-77	Nozzle	4	H <sub>2</sub> O	sub & sat (2 $\phi$ )	p <sub>0</sub> = 17-124	System blowdown experiment
Morrison <sup>9</sup>	10-76	Nozzle	28	H <sub>2</sub> O	sub & sat (2 $\phi$ )	p <sub>0</sub> = 58-67	
Shrock, Starkmann, and Brown <sup>41</sup>	8-76	Nozzle	4-...	H <sub>2</sub> O	sub & sat (2 $\phi$ )	p <sub>0</sub> = 8-91	
Semiscale Project <sup>42</sup>	7-76	Nozzle	13	H <sub>2</sub> O	sub & sat (2 $\phi$ )	p <sub>0</sub> = 3-110	System blowdown experiment
Simoneau <sup>43</sup>	12-75	Nozzle	4	N <sub>2</sub>	sub	p <sub>0</sub> = 5-66	
Semiscale Project <sup>44</sup>	11-75	Nozzle	18	H <sub>2</sub> O	sub & sat (2 $\phi$ )	p <sub>0</sub> = 6-103	System blowdown experiment
∞ Hendricks, Simoneau, and Barrows <sup>45</sup>	9-75	Nozzle	4	N <sub>2</sub>	sub & super- critical	p <sub>0</sub> = 9-102	
Sozzi & Sutherland <sup>12</sup>	7-75	Nozzle	13-76	H <sub>2</sub> O	sub & sat (2 $\phi$ )	p <sub>0</sub> = 30-71	
Dryndrozhik <sup>46</sup>	2-75	Nozzle	6,11	H <sub>2</sub> O	sat (2 $\phi$ )	p <sub>0</sub> = 2-5	
Adachi & Yamamoto <sup>47</sup>	12-74	Nozzle	10	H <sub>2</sub> O	sat (2 $\phi$ )	p <sub>0</sub> = 18-30	
Hendricks, Simoneau, and Ehlers <sup>48</sup>	8-72	Nozzle	3	N <sub>2</sub>	sub & super- critical	p <sub>0</sub> = 12-102	
Deich et al. <sup>49</sup>	4-69	Nozzle	32.5	H <sub>2</sub> O	sat (2 $\phi$ )	p <sub>0</sub> = 1	
Vogrin <sup>50</sup>	7-63	Nozzle	5	Air/H <sub>2</sub> O	simulated 2 $\phi$	p <sub>0</sub> = 1-7	
Neusen <sup>51</sup>	1-62	Nozzle	6,11	H <sub>2</sub> O	sat (2 $\phi$ )	p <sub>0</sub> = 8-65	

Note: cec = conical entrance contour  
 rec = radiused entrance contour  
 sec = sharp entrance contour

sat = saturated liquid state  
 sat (2 $\phi$ ) = saturated two-phase state  
 sub = subcooled state

TABLE 1 Continued

SOURCE	DATE	TYPE	SIZE (mm)		FLUID	REGIME	PRESSURE LEVEL (bars)	COMMENTS
			D	R				
Covelli <sup>52</sup>	1976	orifice	22.5,	30	salt water	sat (2 $\phi$ )	2-4	
Edwards & Jones <sup>15</sup>	1974	orifice	22.5		H <sub>2</sub> O	sat (2 $\phi$ )	p <sub>0</sub> = 2-54	System blowdown experiment
Uchida & Nariai <sup>22</sup>	8-66	orifice	4		H <sub>2</sub> O	sub & sat	p <sub>0</sub> = 0.2-0.8	
Zaloudek <sup>25</sup>	5-63	orifice	13		H <sub>2</sub> O	sub	p <sub>0</sub> = 8	
Friedrich & Vetter <sup>28</sup>	1-62	orifice	4		H <sub>2</sub> O	sub & sat (2 $\phi$ )	p <sub>0</sub> = 6-30	
Friedrich <sup>29</sup>	10-60	orifice	1.5-4		H <sub>2</sub> O	sub & sat (2 $\phi$ )	p <sub>0</sub> = 2-61	
Monroe <sup>53</sup>	1-57	orifice	6-16		H <sub>2</sub> O	sat	p <sub>0</sub> = 2-11	
Pasqua <sup>32</sup>	5-52	orifice	1-3		Freon-12	sub & sat	p <sub>0</sub> 6-9	
Benjamin & Milier <sup>54</sup>	7-41	orifice	6-22		H <sub>2</sub> O	sat	p <sub>0</sub> = 1-21	
Martinez <sup>35</sup>	12-79	globe valve, relief valve	3 4		Freon-11	sub	p <sub>0</sub> = 6-22	
Zaloudek <sup>55</sup>	3-65	tee, elbow	16		H <sub>2</sub> O	sat (2 $\phi$ )	p <sub>e</sub> = 1-6	Stagnation conditions not reported; static pressure near exit constant area section (p <sub>e</sub> ) reported

Note:  $\phi$  = conical entrance contour  
 $\bullet$   $\phi$  = radiused entrance contour  
 $\phi$  = sharp entrance contour

sat = saturated liquid state  
 sat (2 $\phi$ ) = saturated two-phase state  
 sub = subcooled state

TABLE 1 Continued

<u>SOURCE</u>	<u>DATE</u>	<u>TYPE</u>	<u>SIZE</u> (mm)	<u>FLUID</u>	<u>REGIME</u>	<u>PRESSURE</u> <u>LEVEL</u> (bars)	<u>COMMENTS</u>
Fauske & Min <sup>26</sup>	1-63	aperture (9 shapes)	d = 2-7 equivalent	Freon-11	sat	$p_0 = 103$ kPa	
Faletti <sup>56</sup>	12-59	annulus (cec)	d = 5-9 equivalent	H <sub>2</sub> O	sat (2 $\phi$ )	$p_e = 2-7$	Equivalent L/D = 3-107; Stagnation conditions not completely reported; static pressure at exit of constant area section ( $p_e$ ) reported
Moy <sup>31</sup>	1-55	annulus	d = 6-25 equivalent	H <sub>2</sub> O	sat (2 $\phi$ )	$p_e = 27-296$ kPa	Equivalent L/D = 35-96; stagnation conditions not reported; static pressure at exit of area section ( $p_e$ ) reported

cec = conical entrance contour  
 rec = radiused entrance contour  
 sec = sharp entrance contour

sat = saturated liquid state  
 sat (2 $\phi$ ) = saturated two-phase state  
 sub = subcooled state

Sizes are given to the nearest half millimeter. The fluid that was used to perform the tests is indicated in the table. While the majority of the experiments were conducted using water as the test fluid, Freon, nitrogen, and gas-water mixtures were also used. The thermodynamic regime(s) in which the flow stagnation conditions resided is listed in the table as "sub", "sat" or "sat(2 $\phi$ )". These abbreviations denote subcooled conditions, saturated liquid conditions, and saturated two-phase mixture conditions respectively. In order to convey "where the data is" thermodynamically, a range of pressures at which data was recorded is included as a table parameter. It was intended that this parameter would refer to stagnation pressure; however, some of the references did not report stagnation pressures. In these references, the pressure measurement nearest the end of the constant area section was considered of prime importance. Therefore, the range of this pressure has been substituted for these references. A comment section follows the parametric data. These comments provide supplementary information required to adequately describe the experiment or the availability of information.

Several naming conventions have been used in Table 1 to refer to test sections. All test sections containing a constant area section have been designated as "pipes" regardless of size unless the constant area section was both preceded and followed by a varying area section. The type of entrance contour is indicated for each test section that has been designated as a pipe. A 90-degree entrance to the constant area section is indicated by "SEC" denoting sharp entrance contour, a rounded entrance is indicated by "REC" denoting radiused entrance contour, and a conical entrance is indicated by "CEC" denoting conical entrance contour. The exit contour following the constant area duct has not been indicated. Most of the pipes had 90-degree exits. However, some had conical exit contours (e.g., Henry<sup>18,20</sup>, Prisco<sup>13</sup>, Reocreux<sup>5</sup>). The term "nozzle" has been used to denote flow geometries having a varying area section preceding and following the minimum area section. The entrance or diffuser sections may have been conical or of varying radius. The nozzle throat may have been a single cross section or a short constant area section. The term "orifice" has been used to denote flow geometries having a 90-degree entrance and a constant area section having an L/D of 0.1 or less.

## DISCUSSION

The experimental critical flow data inventory presented in Table 1 has several noteworthy aspects. It is clear that the majority of the experimental critical flow data has been obtained using constant area ducts (References 1 through 34). A significant number of critical flow experiments have been conducted using converging-diverging nozzles (References 9, 12, 35 through 51). Data for critical flow occurring in orifices are also available (References 15, 22, 25, 28, 29, 52 through 54), but are quite limited compared to those available for the other two classes of geometry. Only one reference documenting a study of critical flow in tees and elbows (Reference 56) and one study of critical flow in valves (Reference 35) was found. Data on critical flow through geometries resembling a split or crack in a pipe wall or a weldment also seem to be very limited. Only one reference for this type of geometry was found (Reference 26). It is also noteworthy that little of the pipe and nozzle data were obtained using flow geometries that can be considered ideal from the standpoint of an avoidance of flow separation at the entry to the constant area section or throat.

The length and diameter of pipe geometries for which references of experimental studies were found are presented in Figure 1a. Only dimensions of geometries that were tested using water as a test medium are presented. Differences in entrance contour are denoted in Figure 1a by clear symbol to denote data for 90-degree entrances, by using a solid symbol to denote data for rounded entrances, and by using a partially solid symbol to denote data for conical entrances. Figure 1a shows that critical flow data are available for pipes having diameters ranging from 1.5 to 500 mm, and lengths ranging from 0.6 to 2800 mm. It is clear from Figure 1a that a great deal of data are available for test sections having diameters less than or equal to 13 mm. On the other hand, the data from the Marviken CFT Project are the only data that were found for test sections having diameters greater than or equal to 200 mm. In addition, only four experiments were found that were conducted with test sections having diameters between 30 and 200 mm.

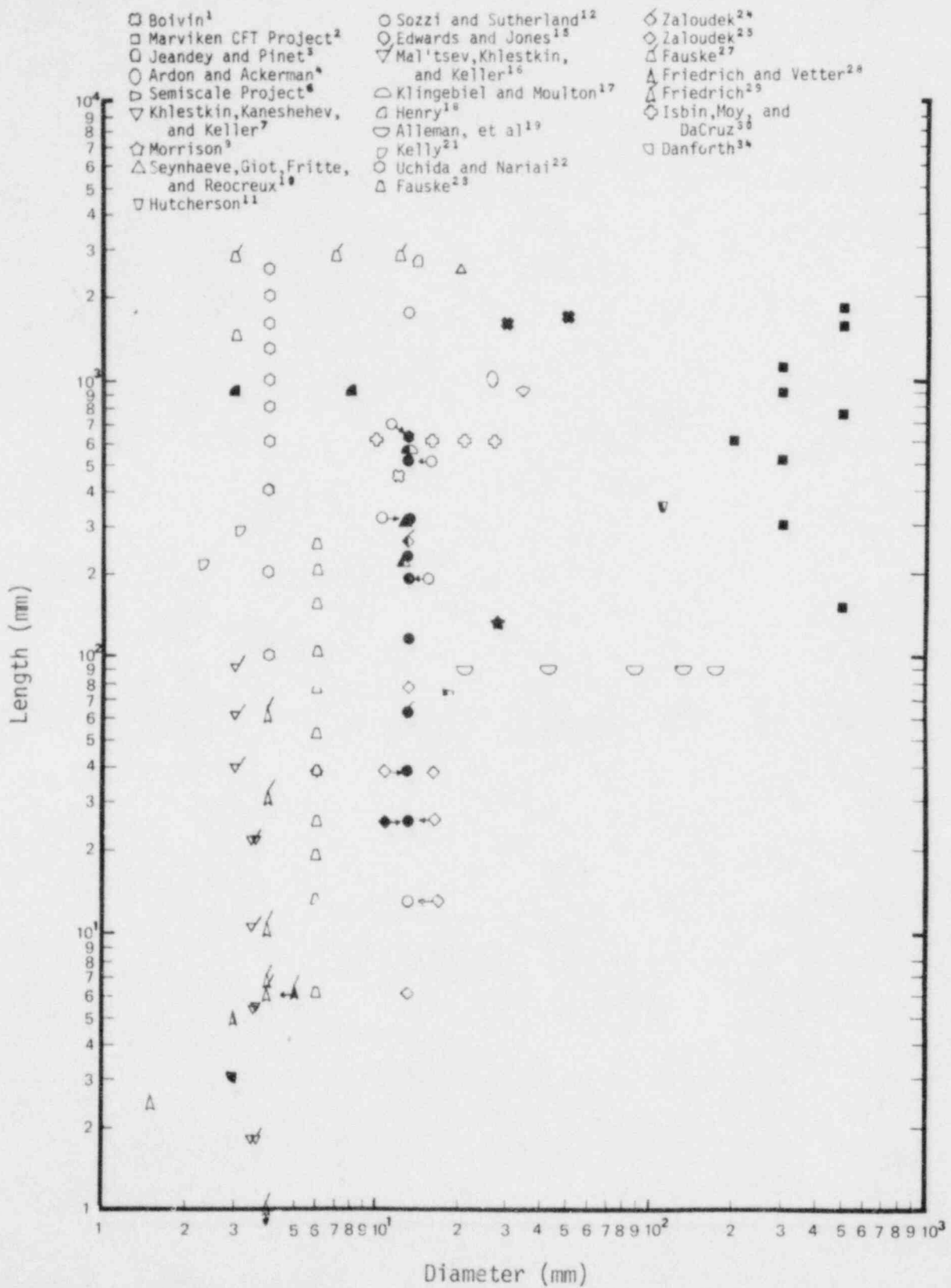


Figure 1a. Lengths and diameters of constant area test sections.

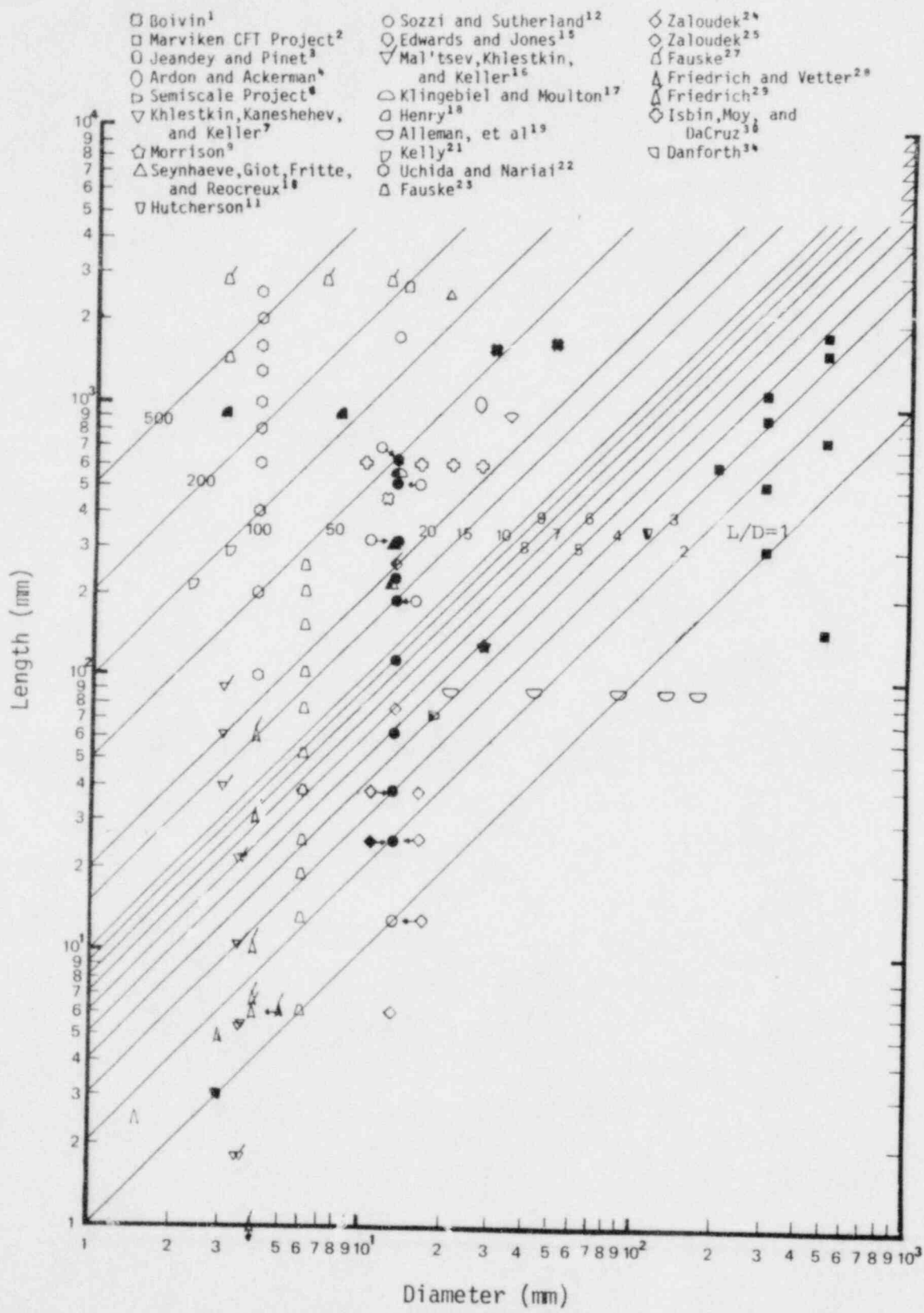


Figure 1b. Length-to-diameter ratio of constant area test sections.

Several studies have been conducted using test sections having the same diameter, but differing in length over a wide range (References 12, 22, and 23). Data are also available for cases in which the test section length was held constant and the diameter was varied (References 19, 27, and 30), although the range of diameter variation was generally more limited than the variation in length at constant diameter in the aforementioned studies.

In order to illustrate the availability of critical flow data in pipes of constant length-to-diameter ratio ( $L/D$ ), lines of constant  $L/D$  have been added to data presented in Figure 1a to produce Figure 1b. This figure shows that data are available for  $L/D$ s ranging from less than 1.0 to over 500. Figure 1b shows that data produced using test sections covering a wide range of size are available at the same  $L/D$  for  $L/D$ s less than four. However, comparing experimental results at the same  $L/D$  would be hampered by differences in entrance contour. Some of the data were obtained using 90-degree entrance contours while other data were obtained using rounded or conical entrance contours. Another factor which would complicate the comparison of data at the same  $L/D$  is that the data are seldom available at the same stagnation conditions.

The throat sizes of converging-diverging nozzles for which experimental critical flow data were found are presented in Figure 2.

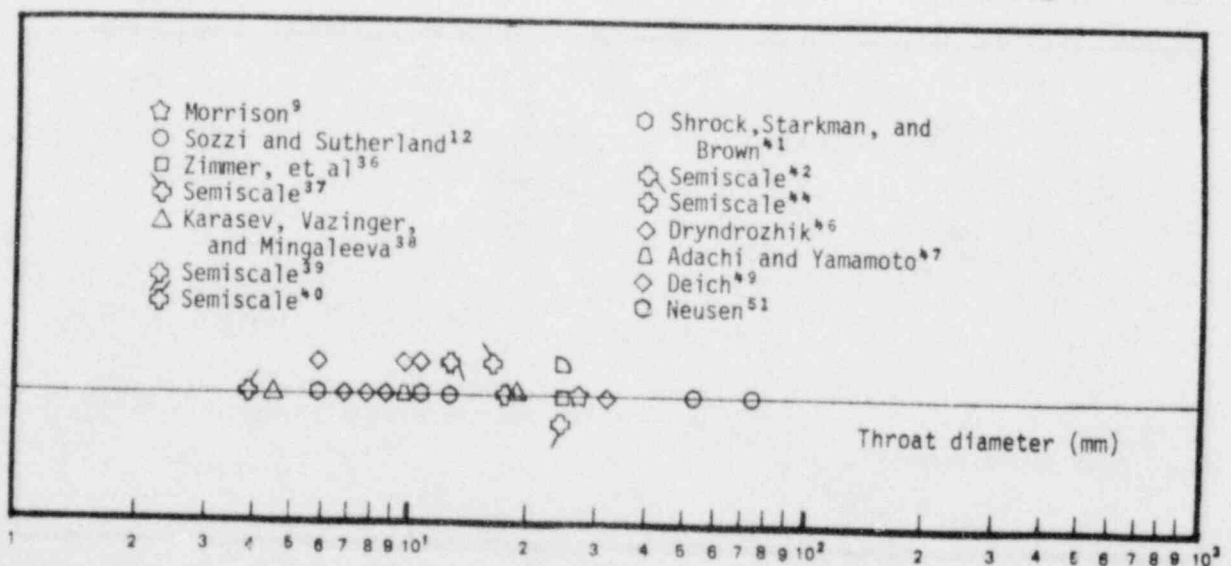


Figure 2. Converging - diverging nozzle throat diameters.



The throat sizes vary from 4 to 75 mm. The data in Figure 2 show that there is little redundancy in size. It is noteworthy that the nozzles vary in entrance contour (conical versus radiused) and in the extent of the minimum area section (a single axial location versus short constant area section).

The sizes of orifices for which experimental critical flow data was found are presented in Figure 3.

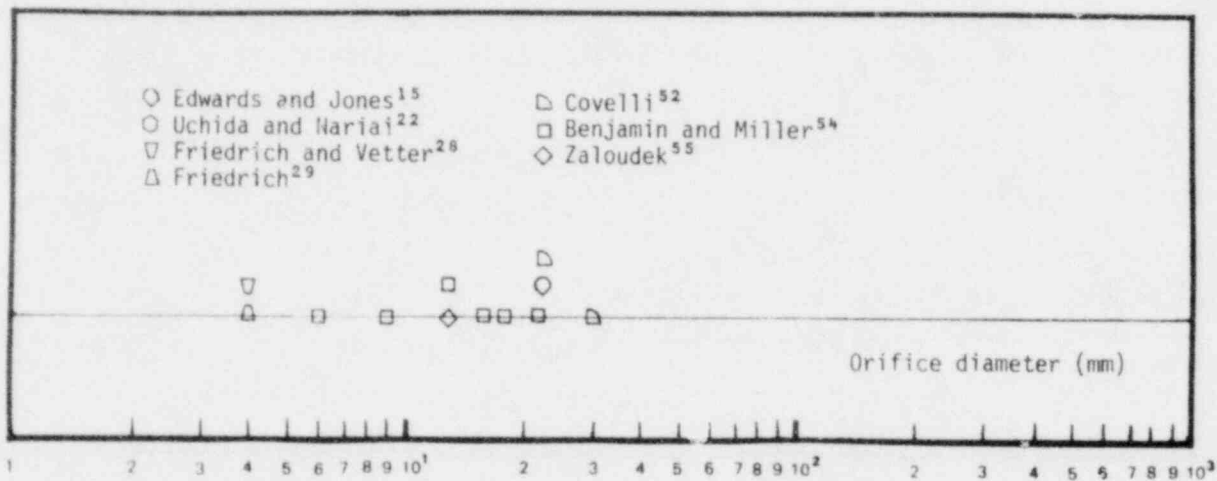


Figure 3. Orifice diameters.

The range of sizes (4 to 30 mm) was quite limited compared to the other two classes of geometries. The orifice size range is reduced to 4 to 22.5 mm if Covelli's data (Reference 52), which were obtained using salt water as the test medium, are not considered. Figure 3 shows that there are only three orifice sizes that have been used in more than one experimental study.

## CONCLUSIONS AND RECOMMENDATIONS

The general conclusions that can be drawn from a review of the critical flow data inventory that has been made are presented in this section. Recommendations for remedying deficiencies in the data base and for improving the design and reporting of future experimental programs are given.

### Conclusions

*A large amount of experimental critical flow data is available.*

Fifty-six documents were found that described experimental critical flow studies and contained experimental data. The studies investigated critical flow in constant area ducts, converging-diverging nozzles, orifices, pipe tees and elbows, valves, and slits.

*The majority of the data was obtained using constant area ducts.*

More than half of the references found documented critical flow studies conducted with constant area ducts. The test sections covered large ranges of diameter (1.5 to 500 mm) and length (0.6 to 2800 mm).

*Significant amounts of critical flow data are available converging-diverging nozzles and orifices over a limited range of sizes.*

Nineteen references were found documenting critical flow studies using converging-diverging nozzles and nine references were found documenting critical flow studies using orifices. The nozzle throat and orifice diameters ranged from 4 to 75 mm and 4 to 30 mm respectively.

*Little data are available for critical flow occurring in standard piping components and for geometries resembling piping failures other than a guillotine break.*

Two references were found in which critical flow was studied in standard plumbing components. These studies utilized small scale elbows, tees, and valves. Only one reference was found in which critical flow was studied in slits simulating a localized pipe failure and again the apparatus used was small scale.

*Little data are available for idealized flow geometries that are designed to avoid entrance separation.*

Most of the constant area ducts had 90-degree, conical, or small radiused entrances (i.e., approximately equal to half the test section diameter). Most of the nozzles had conical entrances of large half-angle and many had an abrupt change in slope at the entrance of the minimum area section and large half-angle diffusers. Very few test sections had gradual approaches to the minimum area section with a continuous change in slope.

*Utilisation of some of the data is hampered by dissimilarities in test section geometry and fluid conditions and by a lack of essential information.*

Differences in test section entrance contour, nozzle throat geometry, and diffuser angle would contribute additional uncertainties if the data were used to assess the effect of geometric variables. Such assessments would also be complicated by a lack of data at common stagnation conditions. Several references did not contain sufficient data to completely specify the stagnation state of flow passing through the nozzle which greatly limits their usefulness for critical flow model assessment and development.

## Recommendations

*Future experimental critical flow studies should be designed to ensure optimal use of and integration with the present data base.*

A large amount of experimental critical flow data is available. This data base should be carefully reviewed as part of planning for future critical flow research to ensure that testing is directed towards expansion of the distribution of geometries for which data is available instead of unnecessary duplication of effort. Test section geometries and stagnation fluid conditions should also be selected to ensure that straightforward comparisons can be made with existing data.

*Tabulated experimental data including a complete specification of the stagnation condition of the flow at the entrance to the test section should be included in the document reporting a critical flow study or a reference to the source from which such data can be obtained should be included.*

The enclosure of tabulated data in the document reporting an experimental critical flow study greatly increases the usefulness of the information. This practice eliminates the need for taking data from report figures. Extracting data from report figures produces data of questionable quality due to possible distortions of the data in producing the plot or in report reproduction, a lack of resolution when using small report figures, and possible errors in reading the data from the plot. Reporting of sufficient data to completely define the stagnation thermodynamic state of the choked flow is important because the flow rate and critical thermodynamic state are primarily a function of the stagnation state. Furthermore, most critical flow models require the stagnation state as input to compute the critical flow rate and critical state. Experimental data which does not include a complete definition of the stagnation state thus cannot be used for model assessment.

*Additional data on critical flow in plumbing components and in pipe failure geometries other than the guillotine break appear to be needed.*

The current interest in small break loss of coolant accidents would seem to increase the need for critical flow data in plumbing components and pipe failures. Modeling of critical flow through safety and relief valves has already been identified as an area of study by the USNRC and other agencies. With the consideration of small breaks, the path to where the flow is being exhausted to the containment may be long and contain numerous plumbing components which are prospective choking locations. The interest in breaks having higher probabilities of occurrence than the guillotine break should increase the interest in critical flow in pipe failure geometries such as pipe splits and weldment cracks.

*The range of sizes of converging-diverging nozzles and orifices for which critical flow data is available is rather limited and may need to be expanded.*

Most of the data for converging-diverging nozzles and orifices have been obtained with test sections having minimum area sections ranging in size from 4 to 20 or 30 mm. Additional data may be required if larger scale applications for hardware from these two classes are identified. The most likely requirement would come from a need to meter flow since these two geometries are typically used in metering devices.

*More complete information on the uncertainties of all measurements made in a critical flow study are needed with particular emphasis on the uncertainty of the measured flow rate.*

Less than half of the references found included measurement uncertainty information. The information that was reported included fluid property measurement uncertainties but generally did not include

the uncertainty of the flow rate measurement. In most cases the uncertainty information was not explained so it could be properly interrupted. Inclusion of measurement uncertainty is essential to determine if significant differences in the data exist that might indicate parametric influences. Measurement uncertainties are also essential in performing analytical model or system code assessment since a clear picture of the uncertainty of the model or code requires knowledge of the uncertainty associated with the data being used to assess the model or code.

*The experimental critical flow data identified in this study should be assembled in a topical data bank to be included in the USNRC Data Bank.*

The utility of the existing experimental data would be greatly increased if it were available in a uniformly formatted form with supporting software for rapid retrieval and data display and manipulation. The USNRC Data Bank currently provides software which allows rapid retrieval and data display and manipulation. Adding of the existing data to the data bank would be a time consuming but not insurmountable task. The benefits of increased understanding of the phenomenon and of improving the state of the art of critical flow prediction justify the effort.

## REFERENCES

1. J. Y. Boivin, "Two-Phase Critical Flow in Long Nozzles," Nuclear Technology, 46, December 1979, pp. 540-545.
2. The Marviken Fullscale Critical Flow Tests; Results of Tests 1-27, MXC201-227.
3. C. H. Jeandey and B. Pinet, "Experimental Study of Critical Two-Phase Flow," OCDE Specialists Meeting on Transient Two-Phase Flow, Paris, France, June 12-14, 1978.
4. K. H. Ardron and M. C. Ackerman, "Studies of the Critical Flow of Subcooled Water in a Pipe," paper presented at the OECD/ CSNI Specialists Meeting on Transient Two-Phase Flow, Paris, France, June 12-14, 1978.
5. M. Reocruz, Contribution to the Study of Critical Flow Rates in Two-Phase Water Vapor Flow, NUREG-TR-0002, August 1977.
6. V. Esparza and K. E. Sackett, Experimental Data Report for Semiscale Mod-1 Test S-06-5 (LOFT Counterpart Test), TREE-NUREG-1125, June 1977.
7. D. A. Khlestkin, V. P. Kanishchev, V. D. Keller, "Flow Characteristics for Hot Water at an Initial Pressure of 22.8 MPa Escaping Into the Atmosphere," Soviet Atomic Energy, 14, 3, 1977, pp. 239-241.
8. M. R. Prisco et al., "Nonequilibrium Discharge of Saturated and Subcooled Liquid Freon-11," Nuclear Science and Engineering, 63, 1977, pp. 365-375.
9. A. F. Morrison, Blowdown Flow in the BWR Test Apparatus, GEAP-21656, October 1977.
10. J. M. Seynhaeve, M. M. Giot, A. A. Fritte, "Nonequilibrium Effects on Critical Flow Rates at Low-Qualities," Specialists Meeting on Transient Two-Phase Flow, Toronto, Canada, August 3-4, 1976.
11. M. N. Hutcherson, Contribution to the Theory of the Two-Phase Blowdown Phenomenon, ANL/RAS 75-42, November 1975.
12. G. L. Sozzi, and W. A. Sutherland, Critical Flow of Saturated and Subcooled Water at High Pressure, NEDO-13418, July 1975.
13. M. R. Prisco, The Nonequilibrium Two-Phase Critical Discharge of Nearly Saturated and Subcooled CCl<sub>3</sub>F Through Short Tubes, ANL-75-9, February 1975.
14. P. A. Howard, One Component Two-Phase Critical Flow: An Experimental Study Using Freon-11 at Subatmospheric Pressures, ANL-75-8, January 1975.

15. A. R. Edwards and C. Jones, An Analysis of Phase IIA Blowdown Tests - The Discharge of High Enthalpy Water from a Simple Vessel into a Containment Volume, SRD R.27, 1974.
16. B. K. Mal'tsev, D. A. Khlestkin, V. D. Keller, "Experimental Investigation of the Discharge of Saturated and Subcooled Water at High Pressures," Thermal Engineering, 19, 6, 1972, pp. 85-88.
17. W. J. Klingebiel, and R. W. Moulton, "Analysis of Flow Choking of Two-Phase, One-Component Mixtures," AIChE Journal, 17, 2, 1971, pp. 383-390.
18. R. E. Henry, An Experimental Study of Low-Quality, Steam-Water Critical Flow at Moderate Pressure, ANL-7740, September 1970.
19. R. T. Alleman et al., Experimental High Enthalpy Water Blowdown from a Simple Vessel Through a Bottom Outlet, BNWL-1411, June 1971.
20. R. E. Henry, A Study of One - and Two-Component, Two-Phase Critical Flows at Low-Qualities, ANL-7430, March 1968.
21. J. T. Kelly, Two-Phase Critical Flow, MS Thesis, Massachusetts Institute of Technology, January 1968.
22. H. Uchida, and H. Nariai, "Discharge of Saturated Water Through Pipes and Orifices," Proceedings of the Third International Heat Transfer Conference, Chicago, Illinois, August 7-16, 1966, pp. 1-12.
23. H. R. Fauske, "The Discharge of Saturated Water Through Tubes," Chemical Engineering Program Symposium Series, 61, 1965, pp. 210-216.
24. F. R. Zaloudek, Steam-Water Critical Flow from High Pressure Systems Interim Report, HW-80535, January 1964.
25. F. R. Zaloudek, The Critical Flow of Hot Water Through Short Tubes, HW-77594, May 1963.
26. H. K. Fauske and T. C. Min, A Study of the Flow of Saturated Freon-11 Through Apertures and Short Tubes, ANL-6667, January 1963.
27. H. K. Fauske, Contribution to the Theory of Two-Phase, One-Component Critical Flow, ANL-6633, October 1962.
28. H. Friedrich and G. Vetter, "The Influence of Nozzle Shapes on Nozzle Flow Behavior for Water at Different Thermodynamic States," Energie, 14, 1, 1957, pp. 373-377.
29. H. Friedrich, "Flow Through Single-Stage Nozzles with Different Thermodynamic States," Energie, 12, 1, 1960, pp. 411-419.



30. H. S. Isbin, J. E. Moy, A. J. R. Da Cruz, "Two-Phase, Steam-Water Critical Flow," AICHE Journal, 3, 3, 1957, pp. 361-365.
31. J. E. Moy, Critical Discharges of Steam-Water Mixtures, MS Thesis, University of Minnesota, January 1955.
32. P. F. Pasqua, Metastable Liquid Flow Through Short Tubes, Ph.D Thesis, Northwestern University, May 1952.
33. J. G. Burnell, "Flow of Boiling Water Through Nozzles, Orifices, and Pipes," Engineering, December 12, 1947, pp. 572-576.
34. J. L. Danforth, Flow of Hot Water Through a Rounded Orifice, BS/MS Thesis, Massachusetts Institute of Technology, May 1941.
35. E. J. Martinec, Two-Phase Critical Flow of Saturated and Subcooled Liquids Through Valves, ANL/RAS/LWR 79-7, December 1979.
36. G. A. Zimmer et al., Pressure and Void Distribution in a converging - Diverging Nozzle with Nonequilibrium Water Vapor Generation, BNL-NUREG 26003, April 1979.
37. R. L. Gillins, K. E. Sackett, K. Stanger, Experimental Data Report for Semiscale Mod-3 Blowdown Heat Transfer Test S-07-3 (Baseline Test Series), NUREG/CR-0356, TREE-I223, December 1978.
38. E. K. Karasev et al., "Investigation of the Adiabatic Expansion of Water Vapor from the Saturation Line in Laval Nozzles," Soviet Atomic Energy, 42, 6, 1977, pp. 545-549.
39. E. M. Feldman, and K. E. Sackett, Experimental Data Report for Semiscale Mod-1 Tests S-05-6 and S-05-7 (Alternate ECC Injection Tests), TREE-NUREG-1055, June 1977.
40. B. L. Collins, H. S. Crapo, and K. E. Sackett, Experimental Data Report for Semiscale Mod-1 Test S-02-6 (Blowdown Heat Transfer Test), TREE-NUREG-1037, January 1977.
41. V. E. Shrock, E. S. Starkman, R. A. Brown, "Flashing Flow of Initially Subcooled Water in Convergent - Divergent Nozzles," ASME-AICE Heat Transfer Conference, St. Louis, MO., August 9-11, 1976, 76-HT-12.
42. H. S. Crapo, M. F. Jensen, K. E. Sackett, Experimental Data Report for Semiscale Mod-1 Test S-29-1 (Integral Test With Symmetrical Break), ANCR-NUREG-1327, July 1976.
43. R. J. Simoneau, Pressure Distribution in a Converging-Diverging Nozzle During Two-Phase Choked Flow of Subcooled Nitrogen, NASA TMX-71762, 1975.

44. H. S. Crapo, M. F. Jensen, K. E. Sackett, Experimental Data Report for Semiscale Mod-1 Test S-02-4 (Blowdown Heat Transfer Test), ANCR-1234, November 1975.
45. R. C. Hendricks, R. J. Simoneau, R. F. Barrows, "Critical Flow and Pressure Ratio Data for LOX Flowing Through Nozzles," Fourteenth International Congress of the International Institute of Refrigeration, Moscow, USSR, September 20-30, 1975 (NASA TMX-71725; E-8347).
46. E. I. Dryndrozhik, "Critical Flow Regime of a Low-Quality Steam-Liquid Mixture in Convergent Nozzles," Fluid Mechanics, 4, 1, 1975, pp. 139-144.
47. H. Adachi, and N. Yamamoto, "High Speed Two-Phase Flow (II), Flashing Flow Through a Converging - Diverging Nozzle," Heat Transfer, 3, 4, 1974, pp. 89-102.
48. R. C. Hendricks, R. J. Simoneau, R. C. Ehlers, "Choked Flow of Fluid Nitrogen with Emphasis on the Thermodynamic Critical Region," Advances in Cryogenic Engineering, 18, 1978, pp. 150-161.
49. M. E. Deich et al., "Investigation of the Flow of Wet Steam in Axisymmetric Laval Nozzles Over a Wide Range of Moisture Content," Teplofizika Vysokikh Temperatur, 7, March-April, 1969, pp. 327-333.
50. J. A. Vogrin Jr., An Experimental Investigation of Two-Phase, Two-Component Flow in a Horizontal, Converging-Diverging Nozzle, ANL-6754, July 1963.
51. K. F. Neusen, Optimizing of Flow Parameters for the Expansion of Very Low-Quality Steam, UCRL-6152, January 1962.
52. B. Covelli, "Critical Flow in Orifices of a Boiling Three-Phase Mixture," Proceedings of the Fifth International Symposium on Fresh Water from the Sea, 2, 1976, pp. 223-232.
53. E. S. Monroe, "Flow of Saturated Boiler Water Through Knife Edge Orifices in Series," Transactions of the ASME, 79, 1, 1957, pp. 373-377.
54. M. W. Benjamin, and J. G. Miller, "The Flow of Saturated Water Through Throttling Orifices," Transactions of the ASME, July 1941, pp. 419-429.
55. F. R. Zalondek, The Low Pressure Critical Discharge of Steam-Water Mixtures from Pipe Elbows and Tees, BNWL-34, March 1965.
56. D. W. Faletti, Two-Phase Critical Flow of Steam-Water Mixtures, Ph.D Thesis, Univeristy of Washington, 1959.

## BIBLIOGRAPHY OF EXPERIMENTAL TWO-PHASE CRITICAL FLOW STUDIES

(Reports included in the experimental data base inventory and presented in Table 1 are indicated by an asterisk.)

- Adachi, H. "High-Speed, Two-Phase Flow, (IV). Two-Phase Discharge Coefficient of Large Sharp-Edged Orifice," Nippon Genshiryoku Gakkaishi, 16, 6, 1974, pp. 322-29.
- \*Adachi, H. and Yamamoto, N., "High Speed Two-Phase Flow (II), Flashing Flow Through A Converging--Diverging Nozzle," Heat Transfer, 3, 4, 1974, pp. 89-102.
- Alyoshin, V. S., Kalaida, Y. A., Fisenko, V. V., "Investigation of Adiabatic Outflow of Saturated and Subcooled Water Through Cylindrical Channels Having Sharp-Edged Inlets," Atomic Energy, 38, 6, 1975, pp. 375-78.
- \*Ardron, K. H., and Ackerman, M. C., "Studies of the Critical Flow of Subcooled Water in a Pipe," paper presented at the OECD/CSNI Specialists Meeting on Transient Two-Phase Flow, Paris, France, June 12-14, 1978.
- Barclay, F. J., Ledwidge, T. J., Cornfield, G. C., Wave Propagation and Critical Flow in Two-Phase, One-Component Mixtures, TRG-1888(D), May 1970.
- Benjamin, M. W. and Miller, J. G., "The Flow of Saturated Water Through Throttling Orifices", Transactions of the ASME, July, 1941, pp. 419-429.
- Bergles, A. E. and Kelly, J. T., "Two-Phase Critical Flow Under Diabatic Conditions," Symposium on Fluid Mechanics and Measurements in Two-Phase Flow Systems, Leeds, England, 1969, Fluid Mechanics and Measurements In Two-Phase Flow Systems, 1969, pp. 129-35.

- Bockh, P. V. and Chwala, J. M., "Propagation Velocity of Pressure Perturbations and Critical Rates of Flow In Gas-Liquid Mixtures," Chem. Ing. Tech., 47, 7, 1975 p. 309.
- \*Boivin, J. Y., "Two-Phase Critical Flow in Long Nozzles," Nuclear Technology, 46, December 1979, pp. 540-545.
- Boure, J. and Reocreux, M., "Two-Phase Critical Flow Rates," Bulletin on Inf. Science and Technology, 197, November 1974, pp. 21-35.
- \*Burnell, J. G., "Flow of Boiling Water Through Nozzles, Orifices, and Pipes", Engineering, December 12, 1947, pp. 572-576.
- \*Collins, B. L., Crapo, H. S., Sackett, K. E., Experimental Data Report for Semiscale Mod-1 Test S-02-6 (Blowdown Heat Transfer Test), TREE-NUREG-1037, January 1977.
- \*Covelli, B., "Critical Flow In Orifices of a Boiling Three-Phase Mixture," Proceedings of the Fifth International Symposium on Fresh Water From the Sea, Alghero, Italy, May 16-20, 1976, 2, pp. 223-232.
- \*Crapo, H. S., Jensen, M. F., Sackett, K. E., Experimental Data Report for Semiscale Mod-1 Test S-29-1 (Integral Test with Asymmetrical Break), ANCR-NUREG-1327, July 1976.
- \*Crapo, H. S., Jensen, M. F., Sackett, K. E., Experimental Data Report for Semiscale Test S--2-4 (Blowdown Heat Transfer Test), ANCR-1234, November 1975.
- \*Danforth, J. L., Flow of Hot Water Through a Rounded Orifice, BS/MS Thesis Massachusetts Institute of Technology, May 1941.
- \*Deich, M. E. et al., "Investigation of the Flow of Wet Steam in Axisymmetric Flow Nozzles Over a Wide Range of Moisture Content," Teplofizika Vysokikh Temperatur, 7, March-April 1969, pp. 327-333.

- \*Drydrozhik, E. I. "Critical Flow Regime in a Low-Quality Steam-Liquid Mixture in Convergent Nozzles," Fluid Mechanics, 4, 1, 1975, pp. 139-44.
- Drydrozhik, E. I., "Critical Velocity of Low-Quality Wet Steam," Journal of Fluid Mechanics, 4, 5, 1975, pp. 33-39.
- \*Edwards, A. R. and Jones, C., An Analysis of Phase IIA Blowdown Tests - The Discharge of High Enthalpy Water From a Simple Volume Into a Containment Volume, SRD-R-27, 1974.
- \*Esparza, V. and Sackett, K. E., Experimental Data Report for Semiscale Mod-1 Test S-06-5 (LOFT Counterpart Test), TREE-NUREG-1125, June 1977.
- \*Faletti, D. W., Two-Phase Critical Flow of Steam-Water Mixtures, Ph.D Thesis, University of Washington, 1959.
- \*Fauske, H. K., Contribution to the Theory of Two-Phase, One-Component Critical Flow, ANL-6633, October 1962.
- \*Fauske, H. K., "The Discharge of Saturated Water Through Tubes," Chemical Engineering Progress Symposium Series, 61, 1965, pp. 210-216.
- Fauske, H. K., "Two-Phase Compressibility in Liquid Metal Systems," International Conference on the Safety of Fast Neutron Reactors, Paris, France, September 1967.
- Fauske, H. K. and Henry, R. E., "On the Measurement of the Throat Pressure in Flashing Sodium Critical Flow," Transactions of the American Nuclear Society, 13, 1, 1970, pp. 330-331.
- \*Fauske, H. K. and Min, T. C., A Study of the Flow of Saturated Freon-11 Through Apertures and Short Tubes, ANL-6667, January 1963.
- Fauske, H. K., Quinn, D. J., Jeans, W. C., "Critical Flow Data of Sodium Liquid-Vapor Mixtures," ANS Transactions, 12, 1, 1969, pp. 354-5.

- \*Feldman, E. M. and Sackett, K. E., Experimental Data Report for Semiscale Mod-1 Tests S-05-6 and S-05-7 (Alternate ECC Injection Tests), TREE-NUREG-1055, June 1977.
- Fox, Z., "Cavitating Venturies and Sonic Nozzles," Proceedings of the National Conference of Fluid Power - 33rd Annual Meeting, October 25-27, 1977, 33, pp. 127-133.
- \*Friedrich, H., "Flow Through Single-Stage Nozzles with Different Thermodynamic States," Energie, 12, 1, 1960, pp. 411-419.
- \*Friedrich, H. and Vetter, G., "The Influence of Nozzle Shape on Nozzle Flow Behavior for Water at Different Thermodynamic States," Energie, 14, 1, 1957, pp. 373-377.
- Grison, P. and Lauro, J. F., "Two-Phase Performance Characteristics of a PWR Primary Pump Under LOCA Conditions," Annual Meeting of the ASME, San Francisco, CA, December 10-15, 1978, pp. 197-212.
- Grison, P. and Lauro, J. F., "Experimental and Theoretical Investigations About Two-Phase Critical Flow Through a Pump," Winter Annual Meeting of the ASME, San Francisco, California, December 10-15, 1978, pp. 197-212.
- Haubenreich, P. N., The Flow of a Flashing Mixture of Water and Steam at High Pressures, CF-55-5-200, May, 1955.
- \*Hendricks, R. C., Simoneau, R. J., Ehlers, R. C., "Choked Flow of Fluid Nitrogen with Emphasis on the Thermodynamic Critical Region," Cryogenic Engineering Conference, Boulder, Colorado, August 9-11, 1972, (NASA-TM-X-68107; E-7044).
- \*Hendricks, R. C., Simoneau, R. J., Barrows, R. F., "Critical Flow and Pressure Ratio Data for LOX Flowing Through Nozzles," Fourteenth International Congress of the International Institute of Refrigeration, Moscow, USSR, September 20-30, 1975, (NASA-TM-X-71725; E-8347).

- \*Henry, R. E., A Study of One- and Two-Component, Two-Phase Critical Flows at Low Qualities, ANL-7430, March 1968.
- Henry, R. E., "Critical Flow of Steam-Water Mixtures," ANS Transactions, 12, 1, 1969, pp. 353-4.
- \*Henry, R. E., An Experimental Study of Low-Quality, Steam-Water Critical Flow at Moderate Pressures, ANL-7740, September 1970.
- Henry, R. E., Fauske, H. K., McComas, S. T., "Two-Phase Critical Flow at Low Qualities - 1,2," Nuclear Science and Engineering, 41, 1, July 1970, pp. 79-98.
- Hillbrath, H. S., "The Critical Flow Venturi, A Useful Device for Flow Measurement and Control," Flow, Its Measurement and Control in Science and Industry, Vol. 1, Pittsburgh, ISA, 1974, pp. 289-97.
- \*Howard, P. A. , One-Component Two-Phase Critical Flow: An Experimental Study Using Freon-11 At Subatmospheric Pressures, ANL-75-8, January 1975.
- \*Hutcherson, M. N., Contribution to the Theory of the Two-Phase Blowdown Phenomenon, ANL/RAS-75-82, December 1975.
- Hutcherson, M. N., Henry, R. E., Wollersheim, D. E., "Experimental Measurements of Large Pipe Transient Blowdown," European Nuclear Conference, Paris, France, April 21-25, 1975.
- Hutcherson, M. N., Henry, R. E., Wollersheim, D. E., "The Two-Phase Blowdown Phenomenon In a Small LWR Geometry," Transactions of the American Nuclear Society 1975 Winter Meeting, San Francisco, California, November 16-21, 1975, 22, pp. 466-7.
- \*Isbin, H. S., Moy, J. E., Da Cruz, A. J. R., "Two-Phase, Steam-Water Critical Flow", AIChE Journal, 3, 3, 1957,, pp. 361-365.

- \*Jeandey, C. H. and Pinet, B., "Experimental Study of Critical Two-Phase Flow," OCDE Specialists Meeting on Transient Two-Phase Flow, Paris France, June 12-14, 1978.
- Ju, Y. H., Use of Laser-Holography Techniques for Studying Two-Phase Steam-Water Critical Flow, Ph. D Thesis, University of Washington, 1977.
- Kalaida, Y. A., Fisenko, V. V., Sychikoh, V. I., "Structure of a Two-Phase Flow in the Critical Cross Section," Thermal Engineering, 23, 3, 1977, pp. 72-73.
- \*Karasev, E. K. et al, "Investigation of the Adiabatic Expansion of Water Vapor from the Saturation Line in Laval Nozzles," Soviet Atomic Energy, 42, 6, June 1977, pp. 545-9.
- Katto, Y. and Sudo, Y., "Study of Critical Flow Completely Separated Gas-Liquid Two-Phase Flow," Bulletin of The JSME, 16, 101, 1973, pp. 1741-1749.
- \*Kelly, J. T. Two-Phase Critical Flow, MS Thesis, Massachusetts Institute of Technology, January 1968.
- \*Khlestkin, D. A., Kanishchev, V. P., Keller, V. D., "Flow Characteristics for Hot Water at an Initial Pressure of 22.8 MPa Escaping Into the Atmosphere," Soviet Atomic Energy, 42, 3, 1977, pp. 239-241.
- \*Klingbiel, W. J. and Moulton, R. W., "Analysis of Flow Choking of Two-Phase, One-Component Mixtures," AICHE Journal, 17, 2, 1971, pp. 383-390.
- Lee, Y. J., Fournay, M. E., Moulton, R. W., "Determination of Slip Ratios in Air-Water Two-Phase Critical Flow at High Quality Levels Utilizing Holographic Techniques," Journal of the AIChE, 20, 2, 1974, pp. 209-219.



- Liles, D. R., Wave Propagation and Choking in Two-Phase Two-Component Flow, Ph.D Thesis, Georgia Institute of Technology, 1974.
- \*Mal'tsev, B. K., Khlestkin, D. A., Keller, V. D., "Experimental Investigation of the Discharge of Saturated and Subcooled Water at High Pressures," Thermal Engineering, 19, 6, 1972, pp. 85-88.
- \*Martinec, E. J., Two-Phase Critical Flow of Saturated and Subcooled Liquids Through Valves, ANL/RAS/LWR 79-7, December 1979.
- \*Monroe, E. S., "Flow of Saturated Boiler Water Through Knife Edge Orifices in Series," Transactions of the ASME, 79, 1, 1957.
- \*Morrison, A. F., Blowdown Flow in the BWR BDHT Test Apparatus, GEAP-21656, October 1977.
- \*Moy, J. E., Critical Discharges of Steam-Water Mixtures, MS Thesis, University of Minnesota, January 1955.
- Nagasaka, H., Kikuchi, O., Aoki, H. "Homogeneous Critical Flow Model Used in Loss-of-Coolant Accident Analysis for BWR," Toshiba Rebyu, 33, 7, 1978, pp. 619-623.
- \*Neusen, K. F., Optimizing of Flow Parameters for the Expansion of Very Low-Quality Steam, UCRL-6152, January 1962.
- Nigmatulin, B. I., Malysenko, V. I., Shugaevz, Y. Z., "Investigation of Liquid Distribution Between Core and the Flim in Annular Dispersed Flow of Steam/Water Mixture", Thermal Engineering, 23, 5, 1977, pp. 66-68.
- Ogasawara, E., Discharge of Saturated Water From Long Ducts: AEC-TR-6928, 1967 (Translated from Nippon Kikai Gakkai Rombunshu, 33, pp. 121-124).

- Palmer, M. E., Sallet, D. W., Wu, K. F., "Influence of Thermodynamic Properties on the Calculation of Homogeneous Mass Flow Rates," ASME Meeting, December 10-15, 1978, 78-WA/HT-48.
- \*Pasqua, P. F. Mestable Liquid Flow Through Short Tubes, Ph.D Thesis, Northwestern University, May 1952.
- \*Prisco, M. R., The Nonequilibrium, Two-Phase Critical Discharge of Nearly Saturated and Subcooled CCl<sub>3</sub>F Through Short Tubes, ANL-75-9, February 1975.
- Prisco, M. R., et al., "Nonequilibrium Critical Discharge of Saturated and Subcooled Liquid Freon-11," Journal of Nuclear Science and Engineering, 63, 4, 1977, pp. 365-375.
- Prisco, M. R., Henry, R. E., Linehan, J. H., "Critical Flow of Saturated Liquid Through Short Tubes," Transactions of the American Nuclear Society, 15, 1, June 1972, pp. 416-17.
- Rassokhin, N. G. et al., "Critical Conditions with Unsteady-State Outflow of a Two-Phase Medium with a Pipeline Break," High Temperature 15, 3, 1977, pp. 491-497.
- Reocreux, M. L., "Experimental Study of Steam-Water Choked Flow," Specialists Meeting on Transient Two-Phase Flow, Toronto, Canada, August 3, 1976, CFA-CONF-3681.
- \*Reocreux, M., Contribution to the Study of Critical Flow Rates in Two-Phase Water Vapor Flow, Vol. 2, Vol. 3, NUREG-TR-0002 (Vol. 2), August 1977, NUREG-TM-0002 (Vol. 3), January 1978.
- \*Seynhaeve, J. M., Gist, M. M., Fritte, A. A., "Nonequilibrium Effects on Critical Flow Rates, at Low-Qualities", Specialists Meeting on Transient Two-Phase Flow, Toronto, Canada, August 3-4, 1976.

\*Shrock, V. E., Starkman, E. S., Brown, R. A., "Flashing Flow of Initially Subcooled Water in Convergent--Divergent Nozzles," ASM-AICHE Heat Transfer Conference, St. Louis, MO., August 9-11, 1976, 76-HT-12 (Journal of Heat Transfer, 99, 2, 1977, pp. 263-268).

Simon, U., "Blowdown Flow Rates of Initially Saturated Water", National Topical Meeting on Water Reactor Safety, Salt Lake City, Utah, March 26-28, 1973, pp. 172-195.

\*Simoneau, R. J., Pressure Distribution in a Converging-Diverging Nozzle During Two-Phase Choked Flow of Subcooled Nitrogen, NASA TMI-71762, 1975.

Smith, R. V., Steam Water, Critical Flow in a Venturi, NBS-TN-608, July 1971.

Smith, R. V., "Two-Phase, Two-Component Critical Flow in a Venturi," Transactions of the ASME, 94 (Ser D), 1, 1972, pp. 147-155.

Smith, R. V., Cousins, L. B., Hewitt, G. F., Two-Phase Two-Component Critical Flow in a Venturi, Aere-R-5736, 1968.

\*Sozzi, G. L. and Sutherland, W. A., Critical Flow of Saturated and Subcooled Water at High Pressure, NEDO-13418, July 1975.

Sudo, Y., "Experimental Study of Homogeneously Dispersed Two-Phased Critical Flow," Bulletin of the JSME, 18, 124, 1975, pp. 1166-1174.

Sudo, Y. and Katto, Y., "Experimental Study of a Completely Separated Two-Component, Two-Phase Critical Flow," Journal of Facility Engineering, 34, 2, September 1977, pp. 315-328.

Tikhonenko, L. K., Kevorkov, L. R., Lutovinov, S. Z., "Investigation of Local Parameters of the Critical Flow of Hot Water in Straight Pipes with a Sharp Leading Edge," Teploenergetika 2 pp. 41-44.

- \*Uchida, H. and Nariai, H., "Discharge of Saturated Water Through Pipes and Orifices," Proceedings of the Third International Heat Transfer Conference, Chicago, Illinois, August 7-16, 1966, pp. 1-12.
- \*Vogrin Jr., J. A., An Experimental Investigation of Two-Phase, Two-Component Flow in a Horizontal, Converging-Diverging Nozzle, ANL-6754, July 1963.
- Wendt, R. E., "Flow: Its Measurement and Control in Science and Industry, Vol. 1," Symposium on Flow, Pittsburgh, PA, May 9, 1971, CONF-710507-P1.
- \*Zaloudek, F. R., Steam-Water Critical Flow from High Pressure Systems Interim Report, HW-80535, January 1964.
- \*Zaloudek, F. R., The Critical Flow of Hot Water Through Short Tubes, HW-77594, May 1963.
- \*Zaloudek, F. R., The Low Pressure Critical Discharge of Steam-Water Mixtures from Pipe Elbows and Tees, BNWL-34, March 1965.
- \*Zimmer, G. A. et al., Pressure and Void Distribution in a Converging-Diverging Nozzle with Nonequilibrium Water Vapor Generation, BNL-NUREG-26003, April 1979.
- Zysin, V. A., Kitanin, E. L., Latypov, F. R., "Critical Efflux of Self-Evaporating Liquid from Cylindrical Channels," Inzh.-Fix.Zh., 32, 1, 1977, (Russian; For English translation see the Journal of Engineering Physics).