UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

May 5, 1980

Docket No. 50-409

LICENSEE: Dairyland Power Cooperative (DPC)

FACILITY: LaCrosse Boiling Water Reactor (LACBWR)

SUBJECT: SUMMARY OF MARCH 19-20, 1980 MEETING TO CLARIFY SELECTED "THREE MILE ISLAND LESSONS LEARNED" ITEMS

On March 19 and 20 NRC representatives and consultants met with DPC representatives at the LACBWR in Genoa, Wisconsin. A list of attendees is attached (attachment 1).

The NRC provided a list of items (attachment 2) as a basis for discussion.

The level of satisfaction in resolving NRC concerns is reflected by the NRC request for further documentation (attachment 3) that in effect, is a summary of the meeting progress as measured against the list of discussion items (attachment 2).

DPC agreed to provide the additional documentation requested by NRC by April 4, 1980.

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Operating Reactors Branch #5
Division of Licensing

Attachments:
As stated

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Attachment 1

LIST OF ATTENDEES
3/19/80 - 3/20/80
LACBWR

John Parkyn	DPC
Lynne Goodman	DPC
C. G. Long	USNRC
G. Lanik	USNRC
J. Donohew	USNRC
O. Chopra	USNRC
J.J. Shea	USNRC
K. R. Ridgway	Region III
L. W. Kelley	DPC
L. Krejiwski	DPC
R. Milos	NES

DISCUSSION ITEMS

2.1.1 a. Describe MDS system and power supply.

b. Describe power supply to level instruments.

2.1.2 Where does BWR owners group stand with valve testing? How will your valves be included?

2.1.3 a. Show us design of direct indication.

b. What plans do you have for additional work an analysis of inadequate core cooling? What is your position on additional insturmentation? Core spray flow?

2.1.4 a. Do all isolated systems isolate on divers parameters?

b. Describe reset modification

c. Does reset affect more than one valve?

d. Is it possible to manually control sample lines following isolation?

e. Why is main steam and feedwater not included here?

2.1.5 a. & c. Describe purging system and modifications, how operated, single failure features, changes in shielding procedures.

2.1.6 a. Why is sampling system not included as high radiation system, also radwaste system? How does this correspond with the essential systems in 2.1.4? North Anna problem.

b. Provide radiation maps for source term. Include required access area of Technical Support Center. Discuss radiation qualification of essential equipment. What shielding mods are required for control room besides minor sheilding between control room and H. P. lab?

2.1.8 a. Where are RCS sample tapes? Are procedures for RCS and containment samples using the current sample stations revised for high rad? Procedures for counting high activity samples? Have you considered backgroung and airborn contamination of counting/analysis facility? Where are present sample stations?

b. Where is existing radiation monitor located? How do you get iodine and particulate samples? Do you have procedures for high radiation samples? Provide data for October 30 letter p. 33, 34, 35.

2.1.8 c. Is the gamma probe a single channel analyser? what cartriges are used? where located?

2.2.1 Show procedures for

a. S. S. responsibilities

b. Shift Technical Advisor

c. Shift turnover procedures

2.2.2 a. C. R. access

b. TSC implementation

c. OpCenter

LA CROSSE BOILING WATER REACTOR

DOCUMENTATION REQUEST

NRC Site Visit 3/19-20/1980

2.1.3b (1) Provide a description of the ongoing program of new instrumentation to detect inadequate core cooling conditions. Modify page 4 of Jan. 31 letter to reflect this action is not complete.

2.1.4 (1) Modify procedures that require the operator to set the valves in closed position prior to reset of the isolation signal.

(2) Commit to perform a review of the overall design of isolation function to determine any modification that could eliminate the above procedural control.

(3) Provide a list of valves on a system basis that has the capability to override an isolation signal.

(4) Provide a list of systems which you might need for potential recovery from an accident. Describe what methods will be used to reopen these systems if the isolation signal sustains.

2.1.5a (1) Revise your Jan. 31 response on pages 8, 9 and 10 that H_2 purge system is not required as part of the licensed bases of your plant.

2.1.6a (1) Provide the measured leak rate data, two weeks after startup from April, 1980 outage, for the off gas system, component cooling water system (i.e. outside containment,) and isolation condenser vent to off gas system. Describe Periodic testing program including sample lines when installed.

(2) Formalize your preventive maintenance (continuous program) to maintain leakage from systems outside containment that could contain radioactivity after an accident to ALAP levels. Describe to the NRC, briefly, this program including list of systems (sampling later when installed,) how you handle ALAP considerations.

(3) Address your work to respond to our letter concerning the North Anna and related incidents problem.

2.1.6b (1) Provide a shielding study of the control room, long term technical support center, long term sampling stations, health physics lab and location for iodine particulate cartridges for effluent lines. Provide any proposed modifications prior to June 1, 1980.

2.1.6b (Continued)

(2) Discuss the radiation qualification of the ESF equipment in the plant. Refer to any documentation that has been previously submitted. If not adequately addressed, provide that information now. Due prior to June 1, 1980.

2.1.8a (1) Document that you have procedures to take and analyze samples (RCS and containment air) which take into account high radiation samples. These samples would be taken if the containment is accessible.

(2) Document where the existing RCS sample taps are located on the RCS and that they will provide a representative sample during an accident. State if these will be the same ones (and only ones) to be used for the new sample station.

(3) Describe off gas high radiation monitor, and its location near the containment, which could be used to monitor radiation from containment and, through procedures, the gross concentration of activity in containment. Describe the means to quantify high activity releases through the off gas system.

2.1.8b (1) Describe the monitor that will be used to quantify high level gross activity being released via the stack during an accident. Provide commitment to install this monitor within two weeks of delivery but no later than prior to startup from the April, 1980 outage.

(2) Provide data requested in Oct. 30 letter, pages 33, 34 & 35.

2.1.8c Describe the equipment in the TSC and the control room to monitor the radioiodine airborne concentration.

2.2.1b (1) Describe the STA short term program, including the manning requirements, qualification and the training being given.

(2) Describe the program for the long term for the STA, including minimum qualification and the training that will be provided by 1/1/81.

(3) Describe the organization that will perform the operational assessment function for both long and short term requirements. Include a description of areas to be covered by this function.

2.2.1c Revise the turnover procedures to include a method for conveying the status of work on the plant being performed by operators, etc. Describe the method to the NRC.

2.2.2b (1) Develop procedures to provide a workable method which will allow you to get plant data from the control room to the TSC now. Describe the method to the NRC.

(2) Expand on your previous proposal of the long term TSC, specifically those concerning the habitability and obtaining plant data from the control room to the TSC.

2.2.2c (1) Establish OSC near the control room for the essential personnel and describe its location. Revise your procedures accordingly.

(2) Describe the use of emergency assembly points as part of the function of OSC.

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