NRC FORM 366 (7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01	Licensee code: I L Q A D 1 (2) — License number: 0 0 0 - 0 0 0 - 0 0 0 (3) — License type: 4 1 1 1 1 (4) — Cat (5)
CON'T 01	Report source: L (6) — Docket number: 0 5 0 0 0 2 5 4 (7) — Event date: 0 4 2 3 8 0 (8) — Report date: 0 5 1 6 8 0 (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 While performing the Loss of Electrohydraulic Control Fluid Pressure Scram
03 surveillance, procedure QIS-39, pressure switch 1-5600-PS-3 was found to trip at
04 896 psig. This is 4 psig below the 900 psig setpoint requirement of TS Table 3.1-3.
05 Since redundant switches were demonstrated to be within allowable limits, an EHC low
06 fluid pressure condition would have been sensed at the proper setpoint and a reactor
07 scram would have been initiated.
08

	System code	Cause code	Cause subcode	Component code	Comp. subcode	Valve subcode
09	I A (11)	E (12)	E (13)	I N S T R U (14)	S (15)	Z (16)

(17) LER/RO Report Number	Event year		Sequential report no.		Occurrence code	Report type		Revision no.
	8 0	—	0 1 0	/	0 3	L	—	0

Action taken	Future action	Effect on plant	Shutdown method	Hours (22)	Attachment submitted	NPRD-4 form sub.	Prime comp. supplier	Component manufacturer
E (18)	Z (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	Y (24)	N (25)	B 0 7 0 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The cause of this occurrence was instrument setpoint drift. The 1-5600-PS-3 pressure
11 switch was adjusted and successfully functionally tested to demonstrate operability.
12
13
14

	Facility status	% Power	Other status (30)	Method of discovery	Discovery description (32)
15	E (28)	0 8 6 (29)	NA	B (31)	Routine Test

	Activity released	Content of release	Amount of activity (35)	Location of release (36)
16	Z (33)	Z (34)	NA	NA

	Personnel exposures number	Type	Description (39)
17	0 0 0 (37)	Z (38)	NA

	Personnel injuries number	Description (41)
18	0 0 0 (40)	NA

	Loss of or damage to facility (43) type	Description
19	Z (42)	NA

	Publicity issued	Description (45)	NRC use only
20	N (44)	NA	

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NAME OF PREPARER S. Kan PHONE: 309-654-2241 Ext. 174

I. LER NUMBER: LER/RO 80-10/03L-0

II. LICENSEE NAME: Commonwealth Edison Company
Quad-Cities Nuclear Power Station

III. FACILITY NAME: Unit One

IV. DOCKET NUMBER: 050-254

V. EVENT DESCRIPTION:

On April 23, 1980 while performing surveillance test procedure QIS-39, Loss of Electrohydraulic Control Fluid Pressure Scram, pressure switch 1-5600-PS-3 was found to trip at 896 psig. This is 4 psig below the 900 psig setpoint requirement of Technical Specification Table 3. 1-3. The other three EHC pressure switches were found to be within the Technical Specification limit. The switch was immediately recalibrated and functionally tested satisfactorily.

VI. PROBABLE CONSEQUENCES OF THE OCCURRENCE:

The four pressure switches are arranged in a one-out-of-two-twice logic to initiate a reactor protection system function upon loss of EHC pressure. Since redundant switches were demonstrated to be within allowable limits an EHC low fluid pressure condition would have been sensed at the proper setpoint and a reactor scram would have been initiated. Safe operation of the reactor was not affected at any time.

VII. CAUSE:

The cause of this occurrence was instrument setpoint drift. The pressure switch is model C-9612-2, manufactured by Barksdale Co.

VIII. CORRECTIVE ACTION:

The immediate corrective action was to recalibrate the pressure switch setpoint. A functional test was then successfully performed to demonstrate the operability of the pressure switch.