NRC FO	RM 366	U. S. NUCLEAR REGULATORY COMMISSION
	LICENSEE E	VENT REPORT
•	CONTROL BLOCK:	(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)
	GAEIIH 2 2000-10000	18ER 25 26 LICENSE TYPE JO 57 CAT 58 5
	REPORT L 6 0 5 0 0 0 3 6 6 6 6 6 0 6 0 0 0 3 6 6 6	0 4 2 8 8 0 8 0 5 1 5 8 0 0 69 EVENT DATE 74 75 REPORT DATE 80
0 2	EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) [At approximately 62% of rated cor	e thermal power during reactor startup,
0 3	the CMFLPD was calculated to be 7	0.3%. Corrective action was taken but j
0 4	Idid not lower the CMFLPD to allow	adjustment of APRMs per Tech Specs
0 5	3.2.2. Preparations were made to	reduce reactor power to less than or
0 6	Lequal to 25% within the next 4 ho	irs, but the problem was corrected bef-j
07	lore any significant reduction. T	nere were no effects on public health j
08	Land safety. This is a repetitive	event - see LER 50-366/1980-010.
0 9 7 8	$\begin{array}{c} \begin{array}{c} \text{CODE} \\ \text{CODE} \\ \text{CODE} \\ \end{array} \begin{array}{c} \text{CODE} \\ \text{CODE} \\ \text{SUBCODE} \\ 10 \end{array} \begin{array}{c} \text{CAUSE} \\ \text{SUBCODE} \\ \text{SUBCODE} \\ 12 \end{array} \begin{array}{c} \text{CAUSE} \\ \text{SUBCODE} \\ 12 \end{array} \begin{array}{c} \text{CAUSE} \\ \text{SUBCODE} \\ 13 \end{array} \begin{array}{c} \text{Z} \\ 13 \end{array}$	COMPONENT CODE SUBCODE SUBCODE $Z Z Z Z Z I 14$ $Z 15$ $Z Z 16$
	17 AEPORT 80 1-1075	CODE TYPE NO.
	ACTION FUTURE EFFECT SHUTDOWN TAKEN ACTION ON PLANT METHOD HOUR	27 28 29 30 31 32 ATTACHMENT NPRD-4 PRIME COMP. COMPONENT
	$\begin{bmatrix} X \\ 33 \end{bmatrix} (B) \\ H \\ 19 \\ 36 \end{bmatrix} (B) \\ Z \\ 20 \\ 36 \end{bmatrix} (Z) \\ 20 \\ 37 \end{bmatrix} (D) \\ 0 \\ 0 \\ 0 \end{bmatrix} (D) \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ $	
[]	CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 2	the servititud on some distribution
10	The high CMFLPD was due to lower	than equilibrium xenon distribution
1 1	Iduring startup. The initial corr	ective action failed because of inef-
12	[fective control rod adjustments.	Personnel were familiarized with bet-
13	I ter, more effective corrective ac	tions to take in similar situations to
14	prevent recurrence.	
	9 FACILITY STATUS SPOWER OTHER STATUS 30 ME DIS L E 1/28 10 1 61 5 1/29 NA I I	B LOUL Engineer observation (32)
7 8	2 10 10 12 13 44 4	5 46 80
1 6 7 8	ELEASED OF RELEASE AMOUNT OF ACTIVITY (35)	45
17	NUMBER TYPE DESCRIPTION (39) NA	
7 8	9 PERSONNEL INJURIES NUMBER DESCRIPTION (41)	03
1 8	0 0 0 0 0 0 NA	
19	LOSS OF OR DAMAGE TO FACILITY (4)	
20	PUBLICITY ISSUED DESCRIPTION (45) N (44) NA	80052806/6 NRC USE ONLY
7 8	NAME OF PREPARED C. L. Coggin, Supt.	68 69 80.5 Plt. Eng. Serv. aucus 912-367-7781
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LER 4: 50-366/1980-075 Licensee: Georgia Power Company Facility Name: Edwin I. Hatch Docket 4: 50-366

Narrative Report for LER 50-366/1980-075

At approximately 2300 CST on 4-27-80, the shift nuclear engineer completed an LPRM-Process Computer calibration (OD-1) and requested a "P-1" calculation. The P1 indicated that the core maximum fraction of limiting power density (CMFLPD) was 70.3% relative to the fraction of rated core thermal power (FRCTP) of 62%. The engineer began taking corrective action within the prescribed 15 minutes to alleviate the CMFLPD problem, but after control rod pattern adjustments at the end of the 2-hour limit the CMFLPD was still too high to allow adjustment of the APRMC as required by Tech Specs section 3.2.2. Preparations were made to have the reactor below 25% of core rated thermal power within the next 4 hours. During this 4-hour period, the CMFLPD was lowered by another control rod pattern adjustment and APRMs adjusted, and normal reactor startup was continued.

To avoid these type occurrences in the future personnel involved have been informed of more effective corrective actions to take in similar situations.

This type event has occurred before and was reported under LER 50-366/1980-010.