



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CONNECTICUT YANKEE ATOMIC POWER COMPANY

DOCKET NO. 50-213

HADDAM NECK PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 35  
License No. DPR-61

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Connecticut Yankee Atomic Power Company (the licensee) dated March 21, 1978, as supported by information submitted by letters dated May 14, 1974, April 18, 1980 and April 23, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

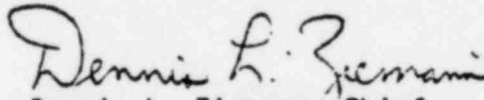
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license Amendment and paragraph 2.C(2) of Operating License No. DPR-61 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B as revised through Amendment No. 35, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Dennis L. Ziemann, Chief  
Operating Reactors Branch #2  
Division of Operating Reactors

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 24, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 35

FACILITY OPERATING LICENSE NO. DPR-61

DOCKET NO. 50-213

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by the captioned amendment number and contain vertical lines indicating the area of changes.

REMOVE PAGES

3-23

3-24

INSERT PAGES

3-23

3-24\*

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\* There are no changes to the provisions contained on this page. The Technical Specifications have merely been repositioned.

### 3.13 REFUELING

Applicability: Applies to operating limitations during refueling operations.

Objective: To insure that no incident could occur during refueling operations that would affect public health and safety.

- Specification:
- A. Radiation levels in the containment and fuel storage building shall be monitored continuously.
  - B. Core subcritical neutron flux shall be continuously monitored by at least two neutron monitors, each with continuous visual and audible indication available, whenever core geometry is being changed. When core geometry is not being changed, at least one neutron flux monitor shall be in service.
  - C. At least one residual heat removal pump shall be in operation.
  - D. During reactor vessel head removal and while loading and unloading fuel from the reactor, the boron concentration shall be maintained at not less than that required to shut down the core to a  $k_{eff}$  - 0.92 (see Specification 3.11)
  - E. One charging pump capable of injecting borated water to the reactor coolant shall be available at all times when changes in core geometry are taking place.
  - F. Whenever new fuel is added to the reactor core, a 1/M plot shall be maintained to verify the subcriticality of the core.
  - G. Direct communication between the control room and the refueling cavity manipulator crane shall be available whenever changes in core geometry are taking place.
  - H. Spent fuel casks shall not be handled above the spent fuel pool or its edge, except as provided in Section 3.13.I, until such time as NRC has received and approved the spent fuel cask drop evaluation.
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- I. After April 23, 1980, a spent fuel cask may be brought into the spent fuel building and may be moved into or over the spent fuel pool a total of ten times in order to remove fuel from the pool for study at an off-site laboratory, or to return the fuel from the laboratory to the pool. Movement of the spent fuel cask under the provisions of this paragraph is conditioned on compliance [by the licensee] with all commitments made by the licensee in its letters to the NRC dated April 18, 1980 and April 23, 1980. In addition, all fuel within the spent fuel pool shall have decayed for at least 90 days before a spent fuel cask is handled above the pool.

Basis:

The equipment and general procedures to be utilized during refueling are discussed in the Facility Description and Safety Analysis. Detailed instructions will be available for use by refueling personnel. These instructions, the above-specified precautions, and the design of the fuel handling equipment incorporating built-in interlocks and safety features, provide assurance that no incident could occur during the refueling operations that would result in a hazard to public health and safety. Whenever no change is being made in core geometry, one flux monitor is sufficient. This permits maintenance of the instrumentation. Continuous monitoring of radiation levels ("A" above) and neutron flux provides immediate indication of an unsafe condition. The residual heat pump is used to maintain a uniform boron concentration. The shutdown margin indicated in Part D will keep the core subcritical, even if all control rods were withdrawn from the core. Weekly checks of refueling water boron concentration insure the proper shutdown margin. Part G allows the control room operator to inform the manipular operator of any impending unsafe condition detected from the main control board indicators during fuel movement.

References:

- (1) FDSA Section 5.2.9
- (2) FDSA Section 7.4