



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

METROPOLITAN EDISON COMPANY

JERSEY CENTRAL POWER AND LIGHT COMPANY

PENNSYLVANIA ELECTRIC COMPANY

DOCKET NO. 50-289

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 54  
License No. DPR-50

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Metropolitan Edison Company, Jersey Central Power and Light Company and Pennsylvania Electric Company (the licensees), dated July 1, 1977, as supplemented and revised August 17 and September 30, 1977, December 13, 1978 and October 26, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

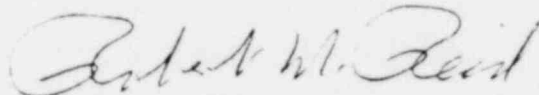
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.c.(2) of Facility Operating License No. DPR-50 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 54, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Operating Reactors

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: May 7, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 54  
FACILITY OPERATING LICENSE NO. DPR-50  
DOCKET NO. 50-289

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
4-11	4-11
4-12	4-12
4-13	4-13
4-13a	4-14
4-14 - 4-27 deleted	
Figure 4.2-1	

The changes on the revised pages are shown by marginal lines.

Applicability

This technical specification applies to the inservice inspection of the reactor coolant system pressure boundary and portions of other safety oriented system pressure boundaries.

Objective

The objective of this inservice inspection program is to provide assurance of the continuing integrity of the reactor coolant system while at the same time minimizing radiation exposure to personnel in the performance of inservice inspections.

Specification

- 4.2.1 Inservice Inspection of ASME Code Class 1, Class 2, and Class 3 components shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a (g), except where specific written relief has been granted by the NRC.
- 4.2.2 The reactor vessel material surveillance capsules removed from TMI-1 during 1976 shall be inserted, irradiated in and withdrawn from the Three Mile Island Unit No. 2 reactor vessel in accordance with the schedule shown in Table 4.2-2. The licensee shall be responsible for the examination of these specimens and for submission of reports of test results in accordance with 10 CFR 50, Appendix H.
- 4.2.3 The accessible portions of one reactor coolant pump motor flywheel assembly will be ultrasonically inspected within 3-1/3 years, two within 6-2/3 years, and all four by the end of the 10 year inspection interval. However, the U.T. procedure is developmental and will be used only to the extent that it is shown to be meaningful. The extent of coverage will be limited to those areas of the flywheel which are accessible without motor disassembly, i.e., can be reached through the access ports. Also, if radiation levels at the lower access ports are prohibitive, only the upper access ports will be used.

4.2.4 The licensee shall submit a report or application for license amendment to the NRC within 90 days after the occurrence of either of the following:

1. Failure of Three Mile Island Unit No. 2 to achieve commercial operation at 100% power by October 1, 1978, or
2. Beginning one year after attainment of commercial operation at 100% power, any time that Three Mile Island Unit No. 2 fails to maintain a cumulative reactor utilization factor of at least 65%.

The report shall provide justification for continued operation of TMI-1 with the reactor vessel surveillance program conducted at Three Mile Island Unit No. 2, or the application for license amendment shall propose an alternative program for conduct for the TMI-1 reactor vessel surveillance program.

For the purpose of this technical specification, the definition of commercial operation is that given in Regulatory Guide 1.16, Revision 4. The definition of cumulative reactor utilization factor is:

Cumulative reactor utilization factor = (Cumulative megawatt hours (thermal) since attainment of commercial operation at 100% power x (100)) divided by (licensed power (MWt) x (Cumulative hours since attainment of commercial operation at 100% power)).

4.2.5 In addition to the reports required by Specification 4.2.4, a report shall be submitted to the NRC prior to September 1, 1982, which summarizes the first five years of operating experience with the TMI-1 integrated surveillance program performed at TMI-2. If, at the time of submission of this report, it is desired to continue the surveillance program at TMI-2, such continuation shall be justified on the basis of the attained operating experience.

#### Bases

- a. Specifications 4.2.1 & 2 ensure that inservice inspection of ASME Code Class 1, 2 and 3 components will be performed in accordance with a periodically updated version of Section XI of the ASME Boiler and Pressure Vessel Code and Addenda as required by 10 CFR 50.55a (g). Relief from any of the above requirements has been provided in writing by the NRC and is not a part of these technical specifications.
- b. Because of damage to the surveillance capsule holder tubes originally installed in TMI-1, irradiation of the TMI-1 capsules will be conducted in TMI-2 pursuant to 10 CFR 50, Appendix H, Section 11.C.4. Because of the similarity of TMI-1 and TMI-2, irradiation in TMI-2 will be substantially equivalent to irradiation in TMI-1, and appropriate adjustments and margins can be imposed in applying the irradiation data to account for such differences as do exist.

The withdrawal schedule has been formulated to optimize the availability of irradiation data from the capsules of both Units 1 and 2.

Because the irradiation program is dependent upon the successful operation at a reasonable utilization of TMI-2, reporting requirements are included to permit reevaluation of the program if TMI-2 does not achieve full power operation in a reasonable period of time or suffers extended outages after the first year of operation.

Pages 4-14 to 4-27 Deleted

Figure 4.2-1 Deleted