

APPLICATION FOR LICENSE TO EXPORT NUCLEAR MATERIAL AND EQUIPMENT (See Instructions on Reverse)

1. APPLICANT'S USE		a. DATE OF APPLICATION 4-17-80		d. APPLICANT'S REFERENCE V-8517		2. NRC USE		a. LICENSE NO. XCAMD401		b. DOCKET NO. 11002044	
3. APPLICANT'S NAME AND ADDRESS						RIS		4. SUPPLIER'S NAME AND ADDRESS (Complete if applicant is not supplier of material)			
a. NAME REUTER-STOKES,								RIS			
b. STREET ADDRESS 18530 SOUTH MILES PARKWAY								a. NAME			
c. CITY CLEVELAND			STATE OHIO		ZIP CODE 44128		b. STREET ADDRESS				
d. TELEPHONE NUMBER (Area Code - Number - Extension) 216-475-3434						c. CITY			STATE		ZIP CODE
5. FIRST SHIPMENT SCHEDULED 90 Days After License		6. FINAL SHIPMENT SCHEDULED 90 Days After License		7. APPLICANT'S CONTRACTUAL DELIVERY DATE 90 Days After License		8. PROPOSED LICENSE EXPIRATION DATE May 1, 1981		9. U.S. DEPARTMENT OF ENERGY CONTRACT NO. (If Known)			
10. ULTIMATE CONSIGNEE						RIS		11. ULTIMATE END USE (Include plant or facility name)			
a. NAME ATOMIC ENERGY OF CANADA LTD.								See attached statement of end use from ultimate consignee.			
b. STREET ADDRESS								11a. EST. DATE OF FIRST USE			
c. CITY - STATE - COUNTRY PINAWA, MANITOBA CANADA								13. INTERMEDIATE END USE			
12. INTERMEDIATE CONSIGNEE						RIS		13. INTERMEDIATE END USE			
a. NAME REUTER-STOKES, CANADA LTD.								Intermediate consignee will receive shipment, but will not use product. He will reship to ultimate consignee.			
b. STREET ADDRESS 465 DOBBIE DRIVE								13a. EST. DATE OF FIRST USE			
c. CITY - STATE - COUNTRY CAMBRIDGE, ONTARIO CANADA								15. INTERMEDIATE END USE			
14. INTERMEDIATE CONSIGNEE						RIS		15. INTERMEDIATE END USE			
a. NAME								15a. EST. DATE OF FIRST USE			
b. STREET ADDRESS											
c. CITY - STATE - COUNTRY											
16. NRC USE	17. DESCRIPTION (Include chemical and physical form of nuclear material; give dollar value of nuclear equipment and components)					18. MAX. ELEMENT WEIGHT		19. MAX. WT. %	20. MAX ISOTOPE WT.	21. UNIT	
	URANIUM 235 DEPOSITED AS UO ₂ ON INTERNAL SURFACE OF REUTER-STOKES MODEL RS-C3-2510-114 RADIATION DETECTOR. EACH DETECTOR CONTAINS 1.4 (GRAMS) URANIUM. FIGURES AT RIGHT ARE FOR THE TOTAL OF ONE DETECTOR TO BE SUPPLIED. VALUE OF EACH DETECTOR IS \$1,370.00 (U.S.)					1.4 grams		93	1.3 grams		
22. COUNTRY OF ORIGIN-SOURCE MATERIAL U.S.A.			23. COUNTRY OF ORIGIN-SNM WHERE ENRICHED OR PRODUCED U.S.A.			24. COUNTRIES WHICH ATTACH SAFEGUARDS (If Known)					
25. ADDITIONAL INFORMATION (Use separate sheet if necessary) Engineering data Sheet 9.08 Attached.											
26. The applicant certifies that this application is prepared in conformity with Title 10, Code of Federal Regulations, and that all information in this application is correct to the best of his/her knowledge. Joseph D. Skarupa											
27. AUTHORIZED OFFICIAL			a. SIGNATURE Joseph D. Skarupa			b. TITLE			8005210128		

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T.
REUTSTOKF CNEG

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J. AGNEW

REQUR WS-19J-69973 END USE AS FOLLOWS:

AS PART OF THE CANADIAN EFFORT IN SAFEGUARDING NUCLEAR REACTORS,
INSTRUMENTS ARE BEING DESIGNED AND TESTED WHICH WILL PROVIDE
IAEA INSPECTORS WITH INDEPENDENT DATA ON A REACTOR'S FUEL INVENTORY.
THE FISSION CHAMBER WILL BE USED AS PART OF A CORE OUTPUT MONITOR
FOR A CANDU REACTOR WHERE IT WILL BE USED TO DETECT NEUTRONS FROM
SPENT FUEL ELEMENTS.

MISS D G PEESKAU
ATOMIC ENERGY OF CANADA LTD
PINAWA MANITOBA

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ATOMIC ENERGY OF CANADA
PINAWA

RS-C3-2510-114 Fission Counter/ Chamber

For Reactor Control (Wide Range)

The RS-C3-2510-114 has proven itself as the standard high-sensitivity fission counter/chamber for wide range reactor instrumentation.

It is designed for measurement of the neutron flux levels from shutdown to full power of nuclear reactor. The detector can be used to detect individual neutrons (counting mode) to 10^6 nv in the presence of an incident gamma flux of 10^6 R/hr.

It can also be used as a wide-range neutron sensor in conjunction with mean-square-voltage (MSV) type circuitry over a range of 10^4 to 10^{10} nv in the presence of an incident gamma flux of 10^6 R/hr.

Operation, as specified here, is greatly dependent on associated electronics. All data presented here is based on measurement using a wide band pre-amplifier such as the model PA-5 manufactured by General Atomic.

Concentric cylinders with uranium coatings provide the neutron sensitive area. Aluminum alloy is used in construction to minimize neutron absorption and residual activity. All seals are directly bonded ceramic to metal. Insulators are high-purity alumina ceramic and are designed to assure stable, long-term noise-free operation of the chambers even at elevated temperature.

This chamber meets the U.S. Specification RDT C15-1T "Fission Type Neutron Detector Assembly" which is part of LMFBR instrumentation development. It can be supplied to the RDT specification which includes integral cable detector housing and cable seals for minimum interference from external noise. The sketch on the back shows this design which is designated Reuter-Stokes model RS-E1-0050.

Another version of this chamber has a 40" sensitive length for core flux averaging in power reactors and is designated model RS-C3-2540-102.

Specifications

MECHANICAL

Maximum diameter.....	8.02 cm
Maximum overall length.....	33.18 cm
Connectors.....	Type HN
Net weight.....	2.4 kg

MATERIAL

Outer shell and inner electrodes.....	1100 Aluminum
Connector.....	6061 Aluminum
Insulation: Detector.....	Alumina ceramic
Connector.....	Alumina ceramic
Neutron sensitive material....	Uranium enriched > 93% in U-235
	Total quantity U-235 = 1.3 gm

CAPACITANCE (See Note 1)

Signal electrode to shell.....	150 pf
HV electrode to shell.....	250 pf

RESISTANCE @ 25° C

Signal electrode to shell.....	10^{13} ohms (minimum)
HV electrode to shell.....	10^{12} ohms (minimum)

MAXIMUM RATINGS

Inter-electrode voltage.....	1000 Volts
Temperature.....	300° C
Burn-up life:	
for 10% decrease in sensitivity.....	3×10^{20} nvt (thermal)

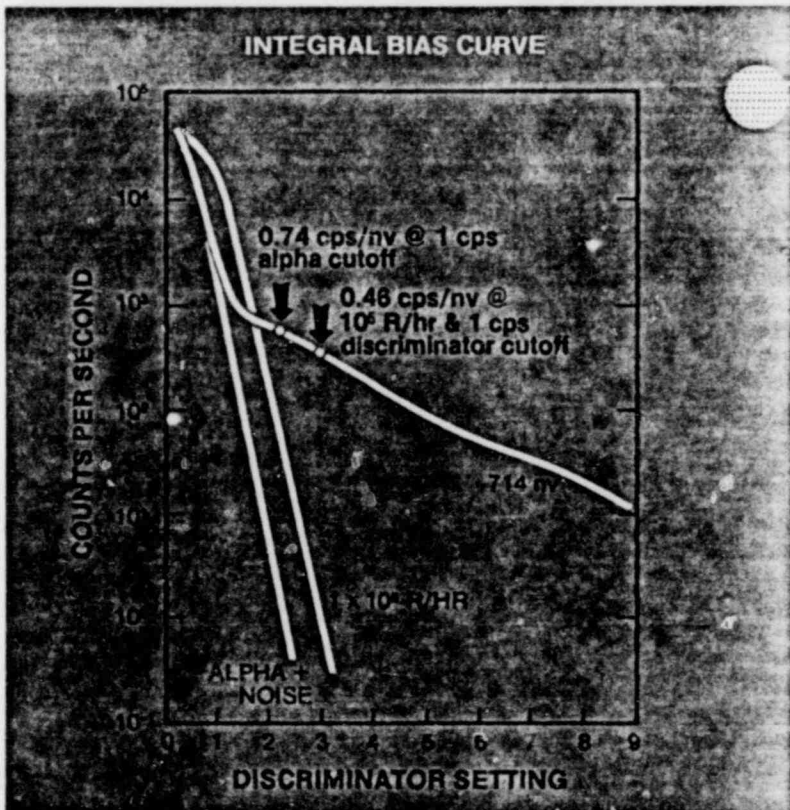
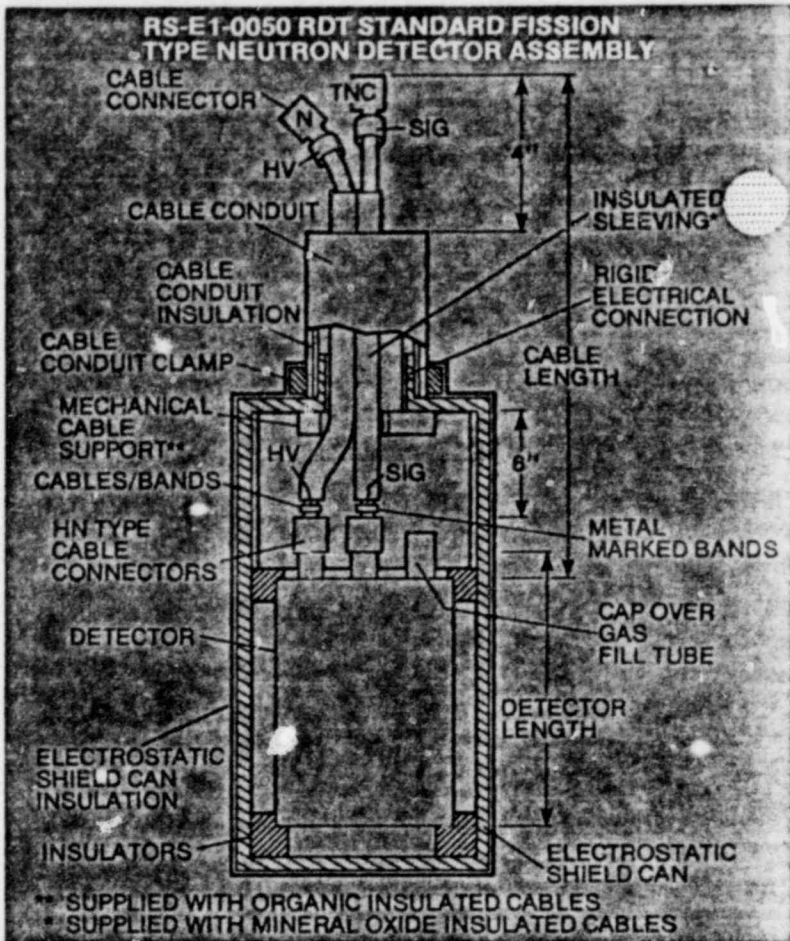
TYPICAL OPERATING CHARACTERISTICS

(See Note 2)

AC thermal neutron sensitivity.....	$> 1 \times 10^{-10} \text{V}^2/\text{nv}$
DC thermal neutron sensitivity.....	$> 1.2 \times 10^{-13} \text{ amp}/\text{nv} \pm 20\%$
Counting sensitivity @ alpha cutoff.....	0.7 cps/nv
AC gamma sensitivity.....	$< 1 \times 10^{-10} \text{V}^2/\text{R}/\text{hr}$
DC gamma sensitivity.....	$< 5 \times 10^{-11} \text{ amp}/\text{R}/\text{hr}$
AC neutron/AC gamma ratio.....	$> 4.5 \text{ R}/\text{hr}/\text{nv}$
DC alpha current.....	$< 8 \times 10^{-9} \text{ amp}$
AC alpha and noise component.....	10^4 nv equivalent
Voltage range.....	400 to 700 volts
Thermal neutron flux range	
in counting mode.....	to 10^6 nv
in MSV mode.....	$> 10^4$ to 10^{10} nv

NOTE 1: With other electrode grounded.

NOTE 2: Operating characteristics are greatly dependent on electronics. All data presented here is based on measurement using a wide band preamplifier such as the model PA-5 manufactured by General Atomic.



reuter ^Rstokes

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