

CONSUMERS POWER COMPANYDOCKET NO. 50-155BIG ROCK POINT PLANTAMENDMENT TO FACILITY OPERATING LICENSEAmendment No. 12
License No. DPP-6

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Consumers Power Company (the licensee) dated March 8, 1976, as modified by letter dated March 3, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

OFFICE ▶						
SURNAME ▶						
DATE ▶						

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Facility License No. DPR-6 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 12, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

original signed

Don K. Davis

Don K. Davis, Acting Chief
Operating Reactors Branch #2
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: MAR 16 1977

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SURNAME →						
DATE →						

ATTACHMENT TO LICENSE AMENDMENT NO. 12

FACILITY OPERATING LICENSE NO. DPR-6

DOCKET NO. 50-155

1. Replace the following pages of the Technical Specifications contained in Appendix A of the above-indicated license with the attached pages bearing the same numbers, except as otherwise indicated. The changed areas on the revised pages are reflected by a marginal line.

Remove

iii
10-2 (Figure 6.2-1)
10-3 (Figure 6.2-1)
10-4 (Table 6.2-1)
10-5
10-7
10-8
10-10
10-11

Insert

iii
10-2 (Figure 6.2-1)
10-3 (Figure 6.2.2)
10-4 (Table 6.2-1)
10-5
10-7
10-8
10-10
10-11

2. Add new Specification 1.1.4 as follows:

Surveillance requirements shall be applicable as specified for individual components or systems as stated in individual specification requirements. Each surveillance requirement shall be performed within the specified time interval with:

- a. A maximum allowable extension not to exceed 25% of the surveillance interval.
- b. A total maximum combined interval time for any three consecutive surveillance intervals not to exceed 3.25 times the specified surveillance interval.

	<u>Page No.</u>
7.0 Operating Procedures	7-1
7.1 Deleted	-
7.2 Deleted	-
7.2.1 Deleted	-
7.2.2 Deleted	-
7.2.3 Deleted	-
7.2.4 Administrative and Procedural Controls Relating to High Performance Fuel	7-3
7.3 Normal Operation	7-3
7.3.1 General	7-3
7.3.2 Cold Start-Up After Extended Shutdown	7-3
7.3.3 Hot Start-Up	7-4
7.3.5 Extended Shutdown	7-5
7.3.6 Short Duration Shutdown	7-6
7.4 Refueling Operation	7-6
7.5 Maintenance (Deleted Except for 7.5.7)	7-8
7.6 Operational Testing of Nuclear Safeguard Systems	7-8
7.7 Deleted	-
8.0 Research and Development Program	8-1
8.1 Developmental Fuel Design Features	8-1
8.1.1 Zr-Cr Alloy Test Bundle	8-1
8.1.2 Thin Clad Power Fuel Bundle	8-5
8.2 Principle Developmental Fuel Operating Limitations	8-5
8.2.1 Center Melt Test Fuel Bundles	8-5
9.0 Primary System Surveillance	9-1
10.0 (Section 6.0) Administrative Controls	10-1
6.1 Responsibility	10-1
6.2 Organization	10-1
6.2.1 Off Site	10-1
6.2.2 Plant Staff	10-1
6.3 Plant Staff Qualifications	10-1
6.4 Training	10-5
6.5 Review and Audit	10-5

Consumers Power Company
OFF-SITE ORGANIZATION

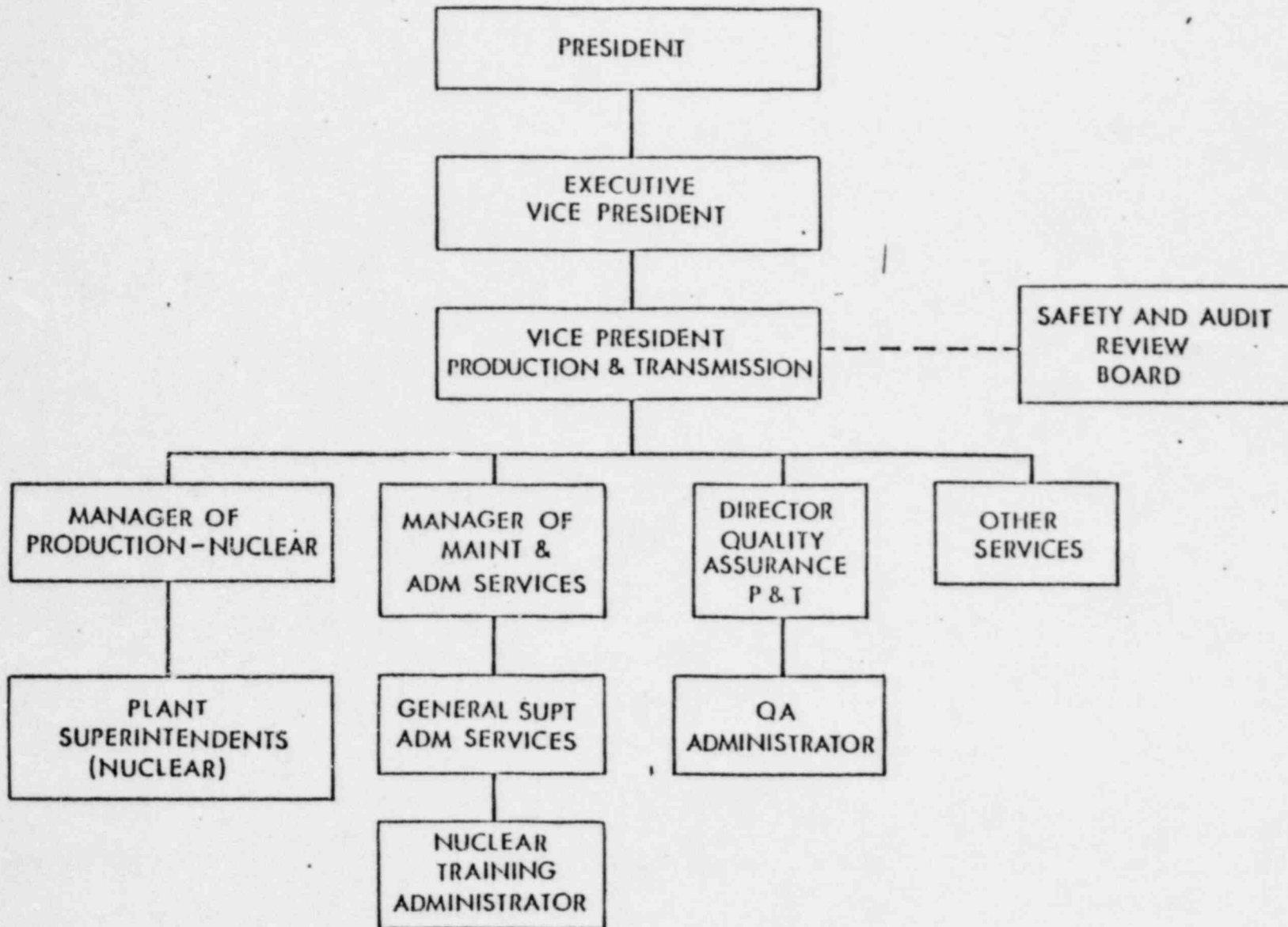
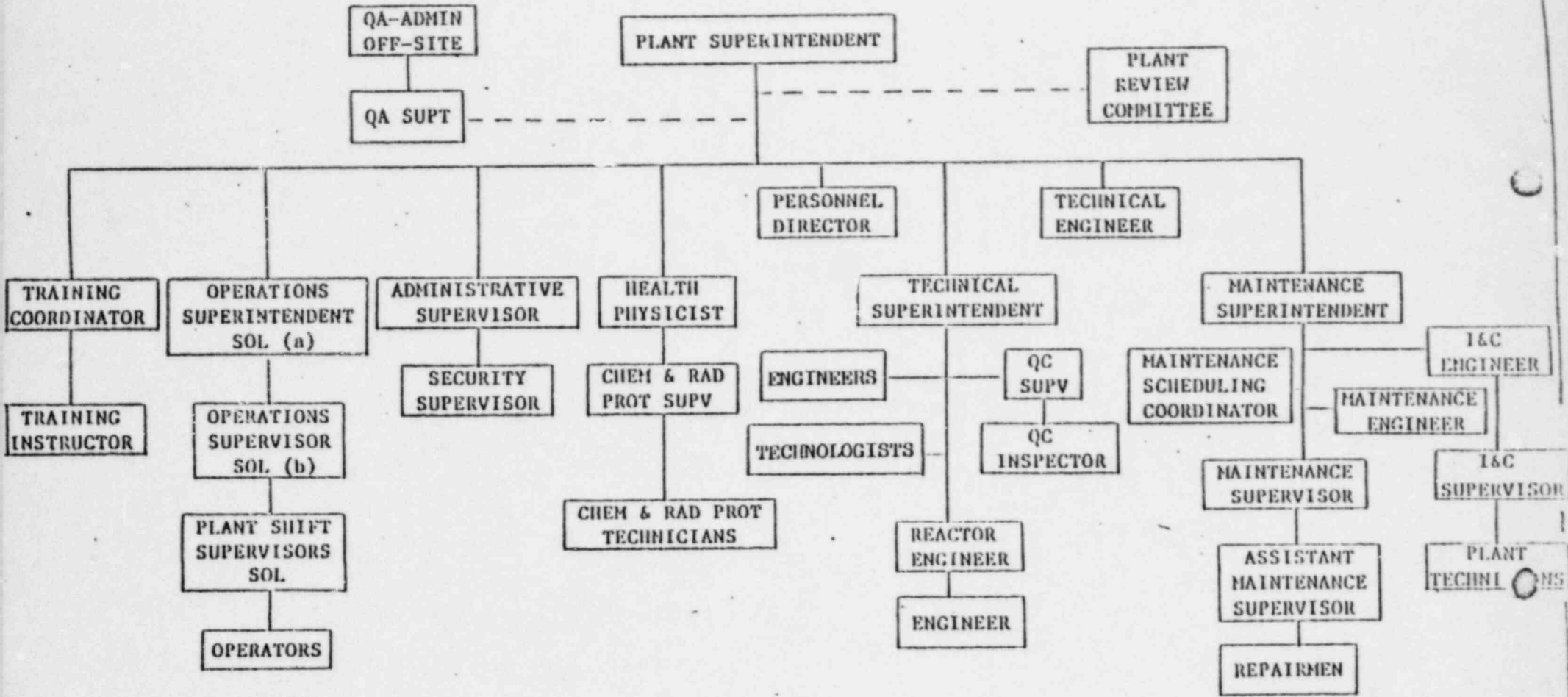


FIGURE 6.2-1

Consumers Power Company
 BIG ROCK POINT
 Organization



SOL - Senior Operator Licensee

- (a) The Operations Superintendent need not hold an SOL provided he meets the other requirements of 6.3.1 of these specifications and the position of "Operations Supervisor" is filled. Under these conditions, the Operations Supervisor shall be responsible for directing the activities of licensed operators.
- (b) This position need not be filled if the Operations Superintendent holds an SOL.

Figure 6.2-2

TABLE 6.2-1

Minimum Shift Crew Composition

The minimum shift crew shall be as follows except when plant conditions specified in paragraph (a) or (b) below have been established (see note (1) below) or when an unexpected absence occurs (see note (2) below):

1 Shift Supervisor - SOL

2 Operators - OL

1 Operator - Nonlicensed

(a) Cold Shutdown

1 Shift Supervisor - SOL

1 Operator - OL

1 Operator - Nonlicensed

(b) Refueling Operations (See note (3) below)

1 Shift Supervisor - SOL

1 Operator - OL

2 Operators - Nonlicensed

SOL - Senior Operator Licensed in accordance with 10 CFR 55.

OL - Operator Licensed in accordance with 10 CFR 55.

- (1) During control rod motion associated with reactor start-up (to a power level \geq 5 percent), one licensed Operator shall observe the control rod manipulation to ensure established control rod withdrawal procedures are adhered to.
- (2) In the event that any member of a minimum shift crew is absent or incapacitated due to illness or injury, a qualified replacement shall report on site within two hours.
- (3) Does not include additional personnel required when core alterations are being conducted. See 6.2.2.e.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the plant staff shall be maintained under the direction of the Nuclear Training Administrator and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR, Part 55.

6.5 REVIEW AND AUDIT

6.5.1 PLANT REVIEW COMMITTEE (PRC)

6.5.1.1 FUNCTION

The Plant Review Committee (PRC) shall function to advise the Plant Superintendent on all matters related to nuclear safety.

6.5.1.2 COMPOSITION

The PRC shall be composed of the:

Chairman: Plant Superintendent
Member: Operations Superintendent
Member: Technical Superintendent
Member: Maintenance Superintendent
Member: Instrument and Control Engineer
Member: Reactor Engineer
Member: Health Physicist
Member: Shift Supervisor
Member: Engineer or Technologist With at Least One-Year Plant Experience

6.5.1.3 ALTERNATES

Alternate members shall be appointed in writing by the PRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in PRC activities at any one time.

6.5.1 (Cont'd)

6.5.1.7 AUTHORITY

The PRC shall:

- a. Recommend to the Plant Superintendent written approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide immediate written notification to the Manager of Production, Nuclear and the Chairman of SARB of disagreement between the PRC and the Plant Superintendent. However, the Plant Superintendent shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

6.5.1.8 RECORDS

The PRC shall maintain written minutes of each meeting and copies shall be provided to the Manager of Production, Nuclear, the Chairman of SARB, PRC members and alternates.

6.5.2 SAFETY AND AUDIT REVIEW BOARD (SARB)

6.5.2.1 RESPONSIBILITIES

SARB is responsible for maintaining a continuing examination of designated plant activities. In all cases, where a matter is formally considered by SARB, its findings and recommendations are communicated in writing to the Vice President - Production & Transmission (P&T) and other appropriate levels of management. A written charter is prepared and approved by the Vice President - P&T which designates the membership, authority and rules for conducting the meetings. Board membership, qualifications, meeting frequency, quorum, responsibilities, authority and records are in accordance with the nuclear plant Technical Specifications and ANSI N18.7-1972.

6.5.2.2 FUNCTION

The SARB shall function to provide independent review of designated activities affecting safety-related components, systems and structures designated on the plant's Safety-Related Quality List contained in the Consumers Power Company Quality Assurance Program.

6.5.2 (Cont'd)

6.5.2.3 COMPOSITION AND QUALIFICATIONS

Collectively, the personnel appointed for the SARB by the Vice President - P&T shall be competent to conduct reviews and technical audits in the following areas:

- a. Nuclear power plant operations.
- b. Nuclear engineering.
- c. Chemistry and radiochemistry.
- d. Metallurgy.
- e. Instrumentation and control.
- f. Radiological safety.
- g. Mechanical and electrical engineering.
- h. Quality Assurance practices.

An individual appointed to the SARB may possess expertise in more than one of the above specialties. He should, in general, have had professional experience at or above the senior engineer level in his specialty.

6.5.2.4 ALTERNATE MEMBERS

Alternate members may be appointed by the Vice President - P&T to act in place of members during any legitimate and unavoidable absences including a conflict-of-interest determination. The qualifications of alternate members shall be similar to those members for whom they will substitute.

6.5.2.5 CONSULTANTS

Consultants shall be utilized as determined by the SARB members and/or chairman to provide expert advice to the SARB. SARB members are not restricted as to sources of technical input and may call for separate investigation from any competent source.

6.5.2 (Cont'd)

- h. The status of deficiencies identified by the Quality Assurance Program, including the effectiveness of the corrective actions completed and implemented, at least once every six (6) months.
- i. Audits of the Security Program required by the "Nuclear Power Plant Security Plan."

6.5.2.9 AUDITS

Audits of safety-related facility activities during operations are performed by the Quality Assurance Department - P&T in accordance with the policies and procedures of the Consumers Power Company Quality Assurance Program. Quality Assurance audit reports are sent to SARB for review. In addition, technical audits are the responsibility of the Technical Services Department and shall be reviewed by SARB. These technical audits encompass:

- a. The conformance of facility operation to all provisions contained within the Technical Specifications and applicable license conditions at least once per year.
- b. The performance, training and qualifications of the entire facility staff at least once per year.
- c. The facility Site Emergency Plan and implementing procedures at least once per two years.
- d. Any other area of facility operation considered appropriate by SARB or the Vice President - P&T.

6.5.2.10 AUTHORITY

SARB shall report to and advise the Vice President - P&T on those areas of responsibility specified in 6.5.2.8 and 6.5.2.9.

6.5.2.11 RECORDS

Records of SARB activities shall be prepared and distributed as indicated below:

- a. Minutes of each SARB meeting shall be prepared and forwarded to the Vice President - P&T and each SARB member within fourteen (14) days following each meeting. Minutes shall be approved at or before the next regularly scheduled meeting following distribution of the minutes.
- b. If not included in SARB meeting minutes, reports of reviews encompassed by Section 6.5.2.8 above shall be prepared and forwarded to the Vice President - P&T within fourteen (14) days following completion of the review.

6.5.2 (Cont'd)

- c. Audit reports encompassed by 6.5.2.9 above, shall be forwarded to the Vice President - P&T and management positions responsible for the areas audited within thirty (30) days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken in the event of a reportable occurrence:

- a. The Commission shall be immediately notified pursuant to 6.7 or a report submitted pursuant to the requirements of 6.9.
- b. All events which are required by regulation or Technical Specifications to be reported to the NRC in writing within 24 hours shall be reviewed by the PRC. The results of the PRC review shall be submitted (either by PRC minutes or by separate report) to SARB and the Manager of Production, Nuclear.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The reactor shall be shut down immediately and not restarted until Commission authorization is received [10 CFR 50.36(c)(1)(1)].
- b. The Safety Limit violation shall be reported immediately to the Commission in accordance with 10 CFR 50.36, the Manager of Production, Nuclear and to SARB Chairman or Vice-Chairman.
- c. A report shall be prepared in accordance with 10 CFR 50.36 and 6.9 of this specification. The Safety Limit violation and the report shall be reviewed by the PRC.
- d. The report shall be submitted within 10 days to the Commission (in accordance with the requirements of 10 CFR 50.36), SARB Chairman and the Manager of Production, Nuclear.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained for all structures, systems, components and safety actions defined in the Big Rock Point Quality List. These procedures shall meet or exceed the requirements of ANSI-18.7.