



UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, D. C. 20555

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May 9, 1980

Honorable John F. Ahearne
Chairman
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: TWO-HUNDRED-FORTY-FIRST MEETING OF THE ADVISORY COMMITTEE ON REACTOR SAFEGUARDS, MAY 1-3, 1980

Dear Dr. Ahearne:

The Advisory Committee on Reactor Safeguards held its 241st meeting in Washington, DC on May 1-3, 1980. Items considered at this meeting are summarized below:

The Committee met with members of the NRC Staff and representatives of the Department of Energy to discuss the proposal to extend operation of the Shippingport Atomic Power Station Light-Water Breeder Reactor core.

The Committee met with members of the NRC Staff and with representatives of the nuclear industry to discuss the proposed Emergency Planning Rule, 10 CFR part 50.

The Committee met with members of the NRC Staff to discuss an acceptable interim approach for dealing with "near-term" construction permit applications. The NRC Staff review of these applications was near completion at the time of the TMI-2 accident.

The Committee has sent you separate reports on the matters noted above.

The Committee has sent you a letter regarding the reduction in the ACRS travel budget.

The Committee met with members of the NRC Staff and with representatives of the Babcock and Wilcox Company to discuss NUREG-0667, Transient Response of B&W-Designed Reactors. Additional time will be scheduled during the 242nd ACRS Meeting to complete this review.

The Committee responded to Commissioner Gilinsky's requests for the Committee's comments regarding clarification of the Committee's report of December 11, 1979 on the "Pause in Licensing", the proposed Nuclear Data Link, and the feasibility and practicality of a containment concept that could withstand a core melt. Copies of these responses have been sent to you.

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The Committee concurred in the proposed regulatory position of Regulatory Guide 1.144 (Rev. 1), Auditing of Quality Assurance Programs for Nuclear Power Plants. It also objected to the NRC Staff procedure to defer assigning numbers to proposed regulatory guides until just prior to their issuance for industry use. The Acting Executive Director for Operations has been advised of these actions.

The Committee determined that it is not necessary to reopen its review of those plants awaiting operating licenses for which it has already written reports regarding operation. The Committee believes that the review of those "NTOL" items required by the NRC Action Plan will provide, on a generic basis, a suitable review for those plants.

The Committee agreed that it does not wish to become more directly involved in matters dealing with the cleanup and decontamination of TMI-2. It does, however, plan to become more directly involved in restoration of the plant for operation.

Members of the NRC Staff briefed the Committee on the cracking of disks (low-pressure) in Westinghouse turbines, the recent jet pump failure at Dresden Nuclear Power Station Unit 3, and on the status of Three Mile Island Nuclear Station Unit 2 cleanup operations.

The Committee met with you to discuss the Committee's report on the NRC Action Plan, the Committee's views regarding its involvement in the TMI-2 decontamination and recovery operations, the status of its report on comparative risk goals, and the Commissioners' needs regarding the ACRS annual report on the NRC's Reactor Safety Research Program budget.

Since the last report of ACRS activities, the following subcommittee meetings have been held:

Evaluation of Licensee Event Reports, in Washington, DC on April 22, 1980

Site Evaluation, in Washington, DC on April 22, 1980

Concrete and Concrete Structures, in Washington, DC on April 22-23, 1980

Reactor Radiological Effects, in Washington, DC on April 23, 1980

Decay Heat Removal Systems, in Washington, DC on April 24, 1980

Metal Components, in Oak Ridge, TN on April 24, 1980

Reactor Fuel, in Washington, DC on April 29, 1980

B&W Water Reactors, in Washington, DC on April 29, 1980

May 9, 1980

Reliability and Risk Assessment, in Washington, DC on April 30, 1980

Regulatory Activities, in Washington, DC on April 30, 1980

Procedures and Administration, in Washington, DC on April 30, 1980

The following items are tentatively scheduled for consideration during the 242nd ACRS Meeting, June 5-7, 1980:

Transient Response of B&W Designed Reactors (NUREG-0667)

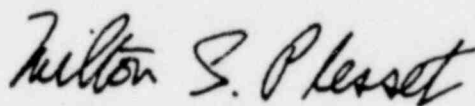
ACRS Report on NRC Reactor Safety Research Program -- FY 1982 Budget
(interim discussion)

Sequoyah Nuclear Power Plant Unit 1 -- Full Power Operating License
(interim review)

Shutdown Decay Heat Removal in Nuclear Power Stations

Systematic Evaluation of Routinely Reported Operating Information

Sincerely yours,



Milton S. Plesset
Chairman