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January 10, 1979

MEMORANDUM FOR: Roger S. Boyd, Director, DPH
Richard C. DeYoung, Director, DSE
Roger J. Mattson, Director, DSS
Victor Stello, Jr., Director, DOR

FROM: Edson G. Case, Chairman
Technical Activities Steering Committee

SUBJECT: SUMMARY OF TECHNICAL ACTIVITIES STEERING
COMMITTEE MEETINGS NUMBERS 12 AND 13,
NOVEMBER 2 AND 13, 1978

At Meeting No. 12 on November 2, 1978, the Steering Committee considered a proposal by its Advisory Group for the definition of an "Unresolved Safety Issue" for use in identifying which issues in NRR's generic issues program qualify for reporting to Congress in accordance with Section 210 of the Energy Reorganization Act of 1974, as amended. The Steering Committee selected the following definition for trial use by the Advisory Group:

"An Unresolved Safety Issue is a matter affecting several nuclear power plants for which it is likely that actions may be needed to (1) compensate for a possible major reduction in the degree of protection of the public health and safety, or (2) provide a potentially significant decrease in risk to the public health and safety, when compared to that expected of plants currently being licensed for construction or operation."

Following Meeting No. 12, the Advisory Group utilized the definition agreed to by the Steering Committee to perform a systematic review

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of the issues in the NRR program. The Advisory Group's recommendations regarding which issues qualify as "Unresolved Safety Issues" were provided to the Steering Committee in a memorandum dated November 9, 1978. A description of the review process utilized by the Advisory Group was also provided in that memorandum. The Steering Committee considered these recommendations of the Advisory Group at Meeting No. 13 on November 13, 1978.

As a result of the Steering Committee's deliberations at Meeting No. 13, the following issues were selected as "Unresolved Safety Issues."

UNRESOLVED SAFETY ISSUES

1. Water Hammer (A-1)
2. Asymmetric Blowdown Loads on the Reactor Coolant System (A-2)
3. Pressurized Water Reactor Steam Generator Tube Integrity (A-3, A-4, A-5)
4. BWR Mark I and Mark II Pressure Suppression Containments (A-6, A-7, A-8, A-39)
5. Anticipated Transients Without Scram (A-9)
6. BWR Nozzle Cracking (A-10)
7. Reactor Vessel Fracture Toughness (A-11)
8. Qualification of Class 1E Safety-Related Electrical Equipment (A-24)
9. Reactor Vessel Pressure Transient Protection (A-26)
10. Residual Heat Removal Requirements (A-31)
11. Seismic Design Criteria (A-40)
12. Pipe Cracks in Boiling Water Reactors (A-42)
13. Emergency Sump Reliability (A-45)
14. Station Blackout (A-44)

The Advisory Group was directed to prepare a staff paper proposing these issues to the Commission. In addition, the staff paper was to include a description of the selection process used by the Steering Committee and a draft Annual Report section describing the progress on each issue.

Original Signed By
E. G. Case

Edson G. Case, Chairman
Technical Activities Steering Committee

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