400 Chestnut Street Tower II

May 5, 1980

Director of Nuclear Reactor Regulation
Attention: Mr. L. S. Rubenstein, Acting Chief
Light Water Reactors Branch No. 4
Division of Project Management
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Mr. Rubenstein:

In the Matter of the Application of) Docket Nos. 50-327 Tennessee Valley Authority) 50-328

Enclosed for your review are two additional requests for relief from ASME Section XI Preservice and Inservice Inspection Requirements for Sequoyah Nuclear Plant pursuant to 10 CFR 50.55a(g)(5). These requests for relief concern the reactor coolant pump casing and the pressureretaining bolting greater than one inch in diameter.

The initial set of requests for relief from ASME Section XI test requirements (in response to S. A. Varga's letter to N. B. Hughes dated December 8, 1978) was transmitted to S. A. Varga by J. E. Gilleland's letter dated February 14, 1979. Amendment 65 will incorporate these requests for relief into question 5.9A of the Sequoyah Nuclear Plant Final Safety Analysis Report as Exemptions 10 and 11.

If you have any questions, please get in touch with D. L. Lambert at FTS 854-2581.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

M. Mills Mana

L. M. Mills, Manager Nuclear Regulation and Safety

Enclosure

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ENCLOSURE

EXEMPTION 10

REQUEST FOR RELIEF

Component: Pressure-retaining bolting

Class:

Inspection Requirements: Visual, surface, and volumetric examination of pressure-retaining belting exceeding one inch

in diameter, examination category C-D.

Basis for Relief: Examination of class 2 pressure-retaining

bolting in accordance with the Summer 1975
Addenda of Section XI exceeds inspection
requirements for class 1 pressure-retaining
bolting. We do not feel an increased level
of safety is obtained from a more restrictive
examination of class 2 components. An examination
program for class 2 pressure-retaining bolting
similar to that for class 1 would be desirable.
This type of examination has been incorporated
in the 1977 Edition, 1978 Addenda of Section XI,
which Sequeyah will be required to meet in the

future.

Alternate Inspection: Class 2 pressure-retaining bolting exceeding

two inches in diameter shall be volumetrically examined in accordance with Table IWC-2500-1, examination category C-D of the 1977 Edition, Summer 1978 Addenda of Section XI. Pressureretaining bolting two inches or less in diameter

will not be examined.

EXEMPTION 11

REQUEST FOR RELIEF

Components: Reactor coolant pumps (four per unir)

Class: 1

Inspection Reduirement: Volumetric examination of pressure-retaining

welds in pump casing, examination category

B-L-1.

Basis for Relief: Each reactor coolant pump casing consists of

a two-piece welded type 304 SST casting. The present capability of ultrasonic tessing is not sufficient to examine cast material of this thickness and achieve meaningful results.

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All four welds will be surface examined for the preservice baseline, and one weld will be

surface examined during each inspection interval.