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(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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REPORT SOURCE

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60	61									68	69						74	75						80
DOCKET NUMBER											EVENT DATE					REPORT DATE								

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

SYSTEM CODE I A 11		CAUSE CODE E 12		CAUSE SUBCODE A 13		COMPONENT CODE I N S T R U 14				COMP. SUBCODE S 15		VALVE SUBCODE Z 16					
EVENT YEAR 8 0 21 22		SEQUENTIAL REPORT NO. 0 3 2 24 26		OCCURRENCE CODE 0 3 28 29		REPORT TYPE L 30		REVISION NO. 0 32									
ACTION TAKEN C 18		FUTURE ACTION A 19		EFFECT ON PLANT Z 20		SHUTDOWN METHOD Z 21		HOURS 0 0 0 0 22		ATTACHMENT SUBMITTED Y 23		NPRD-4 FORM SUB. Y 24		PRIME COMP. SUPPLIER N 25		COMPONENT MANUFACTURER B 0 4 5 26	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

7		8		9		10		11		12		13		14		15		16		17		18		19		20		21		22		23		24		25		26		27		28		29		30		31		32		33		34		35		36		37		38		39		40		41		42		43		44		45		46		47		48		49		50		51		52		53		54		55		56		57		58		59		60	
FACILITY STATUS						% POWER						OTHER STATUS						METHOD OF DISCOVERY						DISCOVERY DESCRIPTION																																																																																			
1		5		X		28		0		7		3		29		Unrelated Trip						A		31		Operator Observation																																																																																	

PERSONNEL EXPOSURES							
NUMBER			TYPE	DESCRIPTION			
1	7		(37) Z	(38)	NA		(39)

8		9		11		12		80	
				LOSS OF OR DAMAGE TO FACILITY					
		TYPE		DESCRIPTION		(43)			
1	9	Z	(42)	NA					

8 9 10 PUBLICITY ISSUED DESCRIPTION (45) NRC USE ONLY

NRC USE ONLY

PHONE: 419-259-5000, Ext. 231

TOLEDO EDISON COMPANY
DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE
SUPPLEMENTAL INFORMATION FOR LER NP-33-80-41

DATE OF EVENT: April 7, 1980

FACILITY: Davis-Besse Unit 1

IDENTIFICATION OF OCCURRENCE: Reactor Protection System (RPS) Channel 2 Trip Module Logic Malfunction

Conditions Prior to Occurrence: The unit was in Mode 1, with Power (MWT) = 2023, and Load (Gross MWE) = 672.

Description of Occurrence: On April 7, 1980 at about 1515 hours, the RPS initiated a high flux trip of the reactor. The trip was caused by work on a feedwater heater which affected condenser vacuum. All four RPS trip modules tripped their respective control rod drive (CRD) trip breakers shutting down the reactor. However, RPS Channel 2's trip module only reflected trip signals from three of the four RPS channels. It did not reflect the trip signal from RPS Channel 3.

The station had entered the action statement of Technical Specification 3.3.1.1 which requires all four channels of RPS high flux trip to be operable in Modes 1 and 2. Following the reactor trip the station was in Mode 3. At 1640 hours on April 7, it was decided to remain shutdown and begin the refueling outage, thus removing the station from the action statement requirements.

Designation of Apparent Cause of Occurrence: The apparent cause of this occurrence was component failure of trip relay K1 in the RPS Channel 2 trip module which energizes upon receipt of a trip signal from RPS Channel 3.

Analysis of Occurrence: There was no danger to the health and safety of the public or to station personnel. RPS Channel 2 tripped its associated CRD trip breaker as designed since only two out of four RPS channel trip signals are required to trip the channel.

Corrective Action: RPS Channel 2's trip module was removed under Maintenance Work Order IC-321-80 and bench tested. Relay K1 was found to be faulty in that its contacts did not always make when the relay was energized. A new trip module was bench tested and installed in RPS Channel 2. ST 5030.12, RPS Trip Module Logic Surveillance Test was performed on Channel 2 and the channel returned to service at 1630 hours on April 8, 1980.

Failure Data: There have been no previous failures of RPS trip modules.

LER #80-032