

Vepco

VIRGINIA ELECTRIC AND POWER COMPANY, RICHMOND, VIRGINIA 23219

May 5, 1980

Mr. James P. O'Reilly, Director
Office of Inspection & Enforcement
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303

Serial No. 539C
PSEGC/GLP:adw:mc

Bracket No. 50-339

Dear Mr. O'Reilly:

In our previous 10CFR50.55(e) submittals concerning the temperature effects on Steam Generator Level Indication during an accident, letters Serial No. 539, 539A and 539B, we indicated that we have completed insulating the reference legs. This was done to insure accurate tripping on Steam Generator Low-Low Level during the initial phases of a Main Steam or Main Feedwater line break inside containment.

We have previously addressed this subject in our response to IE Bulletin 79-21 (Serial No. 678 dated September 14, 1979) and in our response to your request for additional information dated September 23, 1979 (Serial No. 610 dated October 12, 1979 and 310A dated February 25, 1980). These letters deal with the concern for all level measurements inside the containment.

We consider this to be our final report on this item under the provisions of 10CFR50.55(e).

Very truly yours,

Sam C. Brown, Jr.
Senior Vice President
Power Station Engineering
and Construction

cc: Mr. Victor Stello, Director
Office of Inspection & Enforcement

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation

POOR ORIGINAL

B019
SE
/10