INTERIM REPORT

8005120334

Accession No.

Contract Program or Project Title: Advanced Reactor Safety Analysis

Advanced Reactor Safety Analysis Technical Assistance, Reactor Projects

Subject of this Document:

Monthly Highlights for March, 1980

Type of Document:

Monthly Highlights

Author(s):

Robert A. Bari

April 7, 1980

Date of Document:

Responsible NRC Individual and NRC Office or Division:

Dr. Themis P. Speis, Chief Advanced Reactors Branch Division of Project Management U. S. Nuclear Regulatory Commission Washington, D. C. 20555

This document was prepared primarily for preliminary or internal use. It has not received full review and approval. Since there may be substantive changes, this document should not be considered final.

> Brookhaven National Laboratory Upton, New York 11973 Associated Universities, Inc. for the U.S. Department of Energy

Prepared for U.S. Nuclear Regulatory Commission Washington, D.C. 20555 Under Interagency Agreement DE-AC02-76CH00016 FIN No. A-3000

> NRC Research and Technical Assistance Report

for

March 1980*

PROGRAM: Advanced Reactor Safety Analysis Technical Assistance, Reactor Projects Fin No. A-3000

Robert A. Bari, Group Leader

Department of Nuclear Energy BROOKHAVEN NATIONAL LABORATORY Upton, New York 11973

Assistance Report

*Work carried out under the auspices of the United States Nuclear Regulatory Commission.

MONTHLY HIGHLIGHTS

TASK I. ACCIDENT ANALYSIS (R. A. Bari)

Low Heat Flux Sodium Boiling (K. R. Perkins)

Discussions were held with John Meyer (MIT) to aid in the evaluation of Garrison's (ORNL) natural circulation experiment. Issues to be resolved include: loop instabilities, high uncertainty in measured flow rate, and the possibility of damage at low power levels (where superheat problems presented an experimental resolution).

TASK II. POST-ACCIDENT CONTAINMENT ANALYSIS (W. T. Pratt)

FFTF Containment Venting (S. S. Tsai)

A revised memorandum incorporating comments received from H. B. Holz (NRC) was completed and transmitted to DPM/NRC.

TASK III. PLANT DYNAMICS AND RELIABILITY ANALYSIS (R. A. Bari)

FFTF Natural Circulation Test Program (K. R. Perkins)

The two FFTF natural circulation papers presented at the Specialists' Meeting on Decay Heat Removal at BNL have been reviewed. The renewed emphasis exhibited therein on uncertainty analysis (as opposed to regression analysis) will be extremely helpful in evaluating the forthcoming primary loop natural circulation tests.

TASK IV. REACTOR PHYSICS (H. Ludewig)

CDS Study (H. Ludewig)

On April 4, we received specifications for the Phase II CDS core designs from GE. These specifications will form the basis of an analysis of these cores.

DIF-3D (A. Mallen)

Sample problems for the DIF-3D code were received from ANL. Initially, storage problems were experienced in executing these problems. However, these difficulties have been overcome.

Distribution

BNL RSP Associate Chairmen BNL RSP Group Leaders BNL SEG Personnel

J. F. Meyer (15) T. P. Speis U.S. NRC Division of Technical Information and Control (2)