

TABLE 3.2.B

INSTRUMENTATION THAT INITIATES OR CONTROLS THE CORE AND CONTAINMENT
COOLING SYSTEMS

Minimum No. of Operable Instrument Channels Per Trip System (1)	Trip Function	Trip Level Setting	Number of Instrument Channels Provided by Design	Remarks
2	Reactor Low-Low Water Level	≥-48 in. indicated level	4 HPCI & RCIC Inst. Channels	Initiates HPCI & RCIC
2	Reactor Low-Low-Low-Water Level	≥-160 in. indicated level (4)	4 Core Spray & RHR Instrument Channels 4 ADS Instrument Channels	<ol style="list-style-type: none"> 1. In conjunction with Low Reactor Pressure initiates Core Spray and LPCI 2. In conjunction with confirmatory low level High Drywell Pressure, 120 second time delay and LPCI or Core Spray pump interlock initiates Auto Blowdown (ADS) 3. Initiates starting of Diesel Generators

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LIMITING CONDITIONS FOR OPERATION

3.7.A.6.c (Cont'd)

the affected reactor must be taken out of power operation.

- d. A 30 psig limit is the maximum containment repressurization allowable using the CAD system. Venting via the SGBT system to this stack must be initiated at 30 psig following the initial peak pressure of 49.1 psig.

SURVEILLANCE REQUIREMENTS

4.7.A.6.c. (Cont'd)

- ** The CAD system H2 and O2 analyzers shall be tested for operability using standard bottled H2 and O2 once per month and shall be calibrated once per 6 months. The atmosphere analyzing system shall be functionally tested once per operating cycle in conjunction with specification 4.7.A.6.a. Should one of the two H2 or O2 analyzers serving the drywell or suppression pool be found inoperable the remaining analyzer of the same type serving the same compartment shall be tested for operability once per week until the defective analyzer is made operable.

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LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.7.A.7 If the specifications of 3.7.A.1 through 3.7.A.5 cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a cold shutdown condition within 24 hours.

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS3.9.B Operation with Inoperable
Equipment

4.9.B

Whenever the reactor is in Run Mode or Startup Mode with the reactor not in a Cold Condition, the availability of electric power shall be as specified in 3.9.A, except as follows:

1. From and after the date incoming power is not available from one startup or emergency transformer, continued reactor operation is permissible for seven days. During this period, the four diesel generators and associated emergency buses must be demonstrated to be operable.
2. From and after the date that incoming power is not available from both start-up or emergency transformers, continued operation is permissible, provided the four diesel generators and associated emergency buses are operable, all core and containment cooling systems are operable and reactor power level is reduced to 25% of the design.

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS

3.9.B (Cont'd)

4.9.B

3. From and after the date that one of the diesel generators or associated emergency bus is made or found to be inoperable for any reasons, continued reactor operation is permissible in accordance with Specification 3.5.F if Specification 3.9.A.1 is satisfied.
4. From and after the date that one of the diesel generators or associated emergency buses and either the emergency or startup transformer power source are made or found to be inoperable for any reason, continued reactor operation is permissible in accordance with Specification 3.5.F, provided the other off-site source, startup transformer and emergency transformer are available and capable of automatically supplying power to the 4KV emergency buses.
5. From and after the date that one of the 125 volt battery systems is made or found to be inoperable for any reason, continued reactor operation is permissible during the succeeding three days within electrical safety considerations, provided repair work is initiated in the most expeditious manner to return the failed component to an operable state, and Specification 3.5.F is satisfied.

LIMITING CONDITIONS FOR OPERATION3.10 CORE ALTERATIONSApplicability

Applies to the fuel handling and core reactivity limitations.

Objective

To ensure that core reactivity is within the capability of the control rods and to prevent criticality during refueling.

SpecificationA. Refueling Interlocks

1. The reactor mode switch shall be locked in the "Refuel" position during core alterations and the refueling interlocks shall be operable except as specified in 3.10.A.5 and 3.10.A.6 below.
2. Fuel shall not be loaded into the reactor core unless all control rods are fully inserted.

SURVEILLANCE REQUIREMENTS4.10 CORE ALTERATIONSApplicability

Applies to the periodic testing of those interlocks and instrumentation used during refueling and core alterations.

Objective

To verify the operability of instrumentation and interlocks used in refueling and core alterations.

SpecificationA. Refueling Interlocks

1. Prior to any core alterations within or over the reactor core, the reactor switch "Refuel" position interlocks shall be functionally tested. They shall be tested at weekly intervals thereafter until no longer required. They shall also be tested following any repair work associated with the interlocks.
2. Prior to performing control rod or control rod drive maintenance on control cells without removing fuel assemblies, it shall be demonstrated that the core can be made subcritical by a margin of 0.25 percent Δk at any time during the maintenance with the strongest operable control rod

LIMITING CONDITIONS FOR OPERATION

3.10.A Refueling Interlocks

3. The fuel grapple hoist load switch shall be set at ≤ 1000 lbs.
4. If the frame-mounted auxiliary hoist, the monorail-mounted auxiliary hoist, or the service platform hoist is to be used for handling fuel with the head off the reactor vessel, the load limit switch on the hoist to be used shall be set at ≤ 400 lbs.
5. A maximum of two nonadjacent control rods may be withdrawn from the core for the purpose of performing control rod and/or control rod drive maintenance, provided the following conditions are satisfied:
 - a. The reactor mode switch shall be locked in the "refuel" position. The refueling interlock which prevents more than one control rod from being withdrawn may be bypassed for one of the control rods on which maintenance is being performed. All other refueling interlocks shall be operable.
 - b. A sufficient number of control rods shall be operable so that the core can be made subcritical with the strongest operable control rod fully withdrawn and all other operable controls rods fully inserted, or all

SURVEILLANCE REQUIREMENTS

4.10.A.2 (Cont'd)

fully withdrawn and all other operable rods fully inserted. Alternatively if the remaining control rods are fully inserted and have their directional control valves electrically disarmed, it is sufficient to demonstrate that the core is subcritical with a margin of at least $0.25\% \Delta k$ at any time during the maintenance. A control rod on which maintenance is being performed shall be considered inoperable.

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LIMITING CONDITIONS FOR OPERATION

- 3.14.A (Cont'd)
- c. Turbine Building
 - d. Circulating Water Pump Structure
6. When a hose station serving an area which contains equipment which is required to be operable becomes inoperable, establish a continuous fire watch equipped with portable fire suppression equipment within 1 hour and provide equivalent protection to the area served by the inoperable station from the operable hose station within 6 hours.
7. Except as specified in 3.14.A.8 below, the fire suppression spray system serving a Standby Gas Treatment System charcoal filter train shall be operable when a train is required to be operable.
8. If the requirements of 3.14.A.7 cannot be met,
- a. establish a fire water patrol to inspect the area with inoperable fire suppression equipment at least once per shift.
 - b. restore the system to an operable status within 14 days, or in lieu of any other report required by Specification 6.9.2 submit a Special Report to the Commission pursuant to Specification 6.9.3 within 31 days outlining the cause of the malfunction and the plans for restoring the system to an operable status. The SGTS may be considered operable for the purposes of Specification 3.7.B.

SURVEILLANCE REQUIREMENTS

- c. Hose station valve operability and blockage check - once every 3 years.
 - d. Hose hydrostatic test at a pressure at least 50 psig greater than the maximum pressure available at that hose station but not less than 150 psig, or replace with an appropriately tested hose-every 3 years
6. None
7. The SGTS fire suppression spray system testing shall be performed as follows:
- a. Simulated automatic actuation test - once every 18 months
 - b. Inspection of nozzles and spray header- once every 18 months
 - c. Header and nozzle air flow test - once every 3 years

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6.0 ADMINISTRATIVE CONTROLS

6.1 Responsibility

6.1.1 The Station Superintendent shall be responsible for overall facility operation. In the absence of the Station Superintendent, the Assistant Superintendent or the Engineer-Technical (or any other person that the Station superintendent may designate in writing) shall, in that order, assume the Superintendent's responsibility for overall facility operation.

6.2 Organization

Offsite

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

Facility Staff

6.2.2 The facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Figure 6.2-2, except that the shift crew composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements.
- b. At least one licensed operator shall be in the control room and assigned to each reactor that contains fuel.
- c. At least two licensed operators, excluding the operator on the second unit, shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibility during this operation.
- f. A Fire Brigade of at least 5 members shall be maintained onsite at all times. The Fire Brigade shall not include the minimum shift crew necessary for safe shutdown of the unit(s) (3 members) or any personnel required for other essential functions during a fire emergency.

6.5.2 Operation and Safety Review Committee

Function

6.5.2.1 The Operation and Safety Review Committee shall function to provide independent review and audit of designated activities in the area of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. Radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

The members of the OSR Committee will be competent in the area of quality assurance practice and cognizant of the Quality Assurance requirements of 10 CFR 50, Appendix B. Additionally, they will be cognizant of the corporate Quality Assurance Program and will have the corporate Quality Assurance Organization available to them.

Organization

6.5.2.2 The Chairman, Members and alternate Members of the OSR Committee shall be appointed in writing by the Vice President, Electric Production, and shall have an academic degree in an engineering or physical science field; and in addition, shall have a minimum of five years technical experience, of which a minimum of three years shall be in one or more areas given in 6.5.2.1.