

UNITED STATES NUCLEAR REGULATORY COMMISSION
NEGATIVE DECLARATION
REGARDING PROPOSED TECHNICAL SPECIFICATIONS FOR
THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 2
DOCKET NO. 50-320

The U.S. Nuclear Regulatory Commission has determined that the public health, safety and interest require modification of Facility Operating License No. DPR-73, issued to Metropolitan Edison Company, et. al. for operation of the Three Mile Island Nuclear Station Unit 2, located in Londonderry Township, Dauphin County, Pennsylvania. This action would modify the Technical Specifications of the Facility Operating License to more accurately reflect the present condition of the facility resulting from the March 28, 1979 accident. Some of the systems and components currently being used to maintain the facility in its present mode of operation were not originally included in the facility's Technical Specifications. In fact, in the present post-accident status of the facility, the license itself does not include explicit provisions or Technical Specifications for assuring the continued maintenance of the plant in a safe, stable condition or for providing for foreseeable off-normal conditions. Moreover, certain portions of the facility's operating license relate to or govern power operation of the facility, the authority for which was suspended by Order for Modification of License of the Director, Office of Nuclear Reactor Regulation, dated July 20, 1979. These provisions are simply inapplicable to the facility in its present post-accident condition. Consequently, by Order dated February 11,

8002290

302

1980, the Director (1) amended the Order of July 20, 1979 effective immediately, to require that the licensee maintain the facility in accordance with the requirements set forth in Attachment 1 to the Order, and (2) proposed to formally amend the Technical Specifications accordingly, in the following areas: Nuclear Safety; Core Cooling, Water Inventory and Reactor Coolant System Pressure Control; Instrumentation; Containment Systems; Fire Detection and Fire Suppression; Electrical Power; Control of Radioactive Materials in Liquid and Gaseous Effluents; and, Review and Audit Functions.

The Office of Nuclear Reactor Regulation prepared a combined Safety Evaluation and Environmental Assessment (NUREG-0647) in connection with this action. It was determined that since the limits on effluent releases and discharges contained in the environmental Technical Specifications for the Facility Operating License (Appendix B) are not being changed and remain in effect, the actions encompassed by this Order do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. In addition, the Technical Specifications include prohibitions against the purging or other treatment of the reactor building atmosphere, the discharge or other disposal of water decontaminated by the Epicor-II system and the treatment and discharge or other disposal of the high-level radioactively contaminated water now in the reactor building without further Commission approval. Thus, in accordance with this finding, no Environmental Impact Statement will be prepared.

The Safety Evaluation and Environmental Assessment (NUREG-0647) and the Director's Order issued February 11, 1980 are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., and at the Three Mile Island Unit No. 2 Local Public Document Room in the Government Publications Section, State Library of Pennsylvania, Education Building, Commonwealth and Walnut Streets, Harrisburg, Pennsylvania. Copies may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Richard Vollmer, Director TMI-2 Support, NRR.

FOR THE NUCLEAR REGULATORY COMMISSION



Harold R. Denton, Director
Office of Nuclear Reactor Regulation

Dated at Bethesda, Maryland
this 11th day of February, 1980.