



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20566

JUN 09 1978

Those On Attached List

COMPUTER LISTINGS OF LICENSEE EVENT REPORTS SORTED BY FACILITY

The enclosed computer listing provides information concerning licensee event reports entered into the file during the month of May.

If you desire additional information or special searches, please do not hesitate to contact us.

A handwritten signature in cursive script, appearing to read "I. A. Kirk".

I. A. Kirk, Chief
Automated Systems Branch
Division of Technical Support
Office of Management and
Program Analysis

Enclosure:
As stated

8002180031

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LER MONTHLY OUTPUT SORTED BY FACILITY
 PROCESSED DURING MAY FOR POWER REACTORS

FACILITY/SYSTEM/COMPONENT/ COMPONENT SUBCODE/CAUSE CODE/ CAUSE SUBCODE/MANUFACTURER	BUCKET NO./ LER NO./ CONTROL NO.	EVENT DATE/ REPORT DATE/ REPORT TYPE	EVENT DESCRIPTION/ CAUSE DESCRIPTION
DUAD CITIES-2 REAC CORE ISOL COOL SYS + CONT INSTRUMENTATION + CONTROLS SWITCH COMPONENT FAILURE ELECTRICAL LIMITORQUE CORP.	05000265 78-020/031-0 021129	040978 050978 30-DAY	ON APRIL 9, 1978 WHILE PERFORMING RCIC MONTHLY VALVE OPERABILITY TEST QO S 1300-3. IT WAS DETERMINED THAT THE MO-2-1301-16 INBOARD STEAM INLET ISOLATION VALVE WOULD NOT CLOSE. THE MO-2-1301-17 OUTBOARD STEAM ISOLATION VALVE WAS CLOSED AND THE RCIC SYSTEM WAS CONSIDERED INOPERABLE. THE HPCI SYSTEM WAS IMMEDIATELY DEMONSTRATED TO BE OPERABLE, HENCE SAFE PLANT OPERATION WAS UNAFFECTED.
RANCHO SECO-1 TURBINE BYPASS SYS + CONT HANGERS, SUPPORTS, SHOCK, SUPPRESS SNUBBERS COMPONENT FAILURE OTHER BERGEN-PATTERSON PIPE SUPPORT	05000312 78-002/031-0 020877	022878 032378 30-DAY	THIS EVENT WAS DUE TO A FAILED TORQUE SWITCH ON VALVE MO-2-1301-16. THE SWITCH WAS REPLACED AND THE VALVE WAS OPERATED SATISFACTORILY. THE RCIC SYSTEM WAS DECLARED OPERABLE ON APRIL 12, 1978. A MODIFICATION IS BEING INITIATED TO PREVENT INABILITY TO CLOSE THE VALVE WHEN A TORQUE SWITCH FAILS.
RANCHO SECO-1 REACTOR VES. + APPURTENANCES VESSELS, PRESSURE NO SUBCOMPONENT PROVIDED PERSONNEL ERROR LICENSED & SENIOR OPERATORS ITEM NOT APPLICABLE	05000312 78-001/011-0 020579	032078 033178 2-WEEK	TWO SNUBBERS, LIST NOS. 25 AND 26 OF TABLE 4.14-1, WERE FOUND LOW ON FLUID AND REQUIRED REFILLING TO BE RETURNED TO OPERABILITY. THE SNUBBERS HAD PROPER FLUID LEVELS AT THE LAST VISUAL INSPECTION AND FUNCTIONED PROPERLY DURING THE LOCKUP TESTING THAT WAS PERFORMED IN THE REFUELING OUTAGE.
RANCHO SECO-1 COOLANT RECIRC SYS + CONTROLS HANGERS, SUPPORTS, SHOCK, SUPPRESS SNUBBERS OTHER NOT APPLICABLE BERGEN-PATTERSON PIPE SUPPORT	05000312 78-002/031-0 021062	032078 041778 30-DAY	NO EVIDENCE OF LEAKAGE COULD BE DETECTED EITHER ON THE SNUBBERS OR ON ANY COMPONENT NEAR THE SNUBBERS. GIVEN THE HISTORY OF THE SNUBBERS, NO CAUSE COULD BE DETERMINED. THE SNUBBERS WERE REFILLED AND, THE SURVEILLANCE INTERVAL HAS BEEN REDUCED TO SIX MONTHS.
			AN OPERATOR CHANGING A LIGHTBULB IN A BACKLIGHTED PUSHBUTTON IN THE CONTROL ROOM, SHORTED OUT DC POWER TO THE PLANT NON-NUCLEAR INSTRUMENTATION. LOSS OF APPROXIMATELY 2/3 OF THE TEMPERATURE, PRESSURE, FLOW AND LEVEL SIGNALS RESULTED IN A REACTOR TRIP AND SUBSEQUENT RCS COOLDOWN AT 300 DEGREES PER HOUR. THIS RATE EXCEEDED LIMITS ESTABLISHED IN TECH SPECS FIGURE 3.1.2-2. THE REACTOR VENDOR ANALYZED THE TRANSIENT AND IMPOSED CONDITIONS FOR A RETURN TO POWER.
			THE LOSS OF NNI CAUSED INVALID SYSTEMS TO BE DISPLAYED IN THE CONTROL ROOM AND USED AS INPUTS TO THE ICS. FOLLOWING THE REACTOR TRIP, LACK OF RCS TEMPERATURE INDICATION RESULTED IN RAPID COOLDOWN BECAUSE OPERATORS ALLOWED AUXILIARY FEEDWATER TO RUN UNTIL NNI WAS RESTORED.
			IN ACCORDANCE WITH TECHNICAL SPECIFICATIONS SECTION 4.14.1, THE PERIODIC VISUAL INSPECTION OF "INACCESSIBLE" SAFETY SYSTEM HYDRAULIC SNUBBERS WAS PERFORMED ON MARCH 20, 1978. ONE SNUBBER, LIST NO. 55-20 TABLE 4.14-1, WAS FOUND LOW ON FLUID AND REQUIRED REFILLING TO BE RETURNED TO OPERABILITY. THE SNUBBER HAD A PROPER FLUID LEVEL AT THE LAST VISUAL INSPECTION AND NO OBVIOUS OIL LEAKAGE WAS PRESENT.
			NO EVIDENCE OF LEAKAGE COULD BE DETECTED EITHER ON THE SNUBBER OR ON ANY COMPONENT NEAR THE SNUBBER. GIVEN THE HISTORY OF THE SNUBBER, NO CAUSE COULD BE DETERMINED. THE SNUBBER WAS REFILLED AND, IN ACCORDANCE WITH TECHNICAL SPECIFICATIONS SECTION 4.14.1, THE SURVEILLANCE INTERVAL HAS BEEN INCREASED TO TWELVE MONTHS.