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DIVISION 1 REGULATORY GUIDES
POWER REACTORS

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Number	Title	Rev.	Issued Year/Month
1.1	Net Positive Suction Head for Emergency Core Cooling and Containment Heat Removal System Pumps (Safety Guide 1)	---	70/11
1.2	Thermal Shock to Reactor Pressure Vessels (Safety Guide 2)	---	70/11
1.3	Assumptions Used for Evaluating the Potential Radiological Consequences of a Loss-of-Coolant Accident for Boiling Water Reactors	--- 1 2	70/11 73/06 74/06
1.4	Assumptions Used for Evaluating the Potential Radiological Consequences of a Loss-of-Coolant Accident for Pressurized Water Reactors	--- 1 2	70/11 73/06 74/06
1.5	Assumptions Used for Evaluating the Potential Radiological Consequences of a Steam Line Break Accident for Boiling Water Reactors (Safety Guide 5)	---	71/03
1.6	Independence Between Redundant Standby (Onsite) Power Sources and Between Their Distribution Systems (Safety Guide 6)	---	71/03
1.7	Control of Combustible Gas Concentrations in Containment Following a Loss-of-Coolant Accident	--- 1 2	71/03 76/09 78/11
1.8	Personnel Selection and Training	--- 1 1-R	71/03 75/09 77/05
1.9	Selection, Design, and Qualification of Diesel-Generator Units Used as Onsite Electric Power Systems at Nuclear Power Plants (For Comment)	--- 1	71/03 78/11
1.10	Mechanical (Cadmeld) Splices in Reinforcing Bars of Category 1 Concrete Structures	--- 1	71/03 73/01

Number	Title	Rev.	Issued Year/Month
1.11	Instrument Lines Penetrating Primary Reactor Containment (Safety Guide 11) Supplement to Safety Guide 11, Backfitting Considerations	---	71/03 72/02
1.12	Instrumentation for Earthquakes	---	71/03
		1	74/04
1.13	Spent Fuel Storage Facility Design Basis (For Comment)	---	71/03
		1	75/12
1.14	Reactor Coolant Pump Flywheel Integrity (For Comment)	---	71/10
		1	75/08
1.15	Testing of Reinforcing Bars for Category I Concrete Structures	---	71/10
		1	72/12
1.16	Reporting of Operating Information--Appendix A Technical Specifications (For Comment)	---	71/10
		1	73/10
		2	74/09
		3	75/01
		4	75/08
1.17	Protection of Nuclear Power Plants Against Industrial Sabotage	---	71/10
		1	73/06
1.18	Structural Acceptance Test for Concrete Primary Reactor Containments	---	71/10
		1	72/12
1.19	Nondestructive Examination of Primary Containment Liner Welds (Safety Guide 19)	---	71/12
		1	72/08
1.20	Comprehensive Vibration Assessment Program for Reactor Internals During Preoperational and Initial Startup Testing	---	71/12
		1	75/06
		2	76/05
1.21	Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants	---	71/12
		1	74/06
1.22	Periodic Testing of Protection System Actuation Functions (Safety Guide 22)	---	72/02
1.23	Onsite Meteorological Programs (Safety Guide 23)	---	72/02
1.24	Assumptions Used for Evaluating the Potential Radiological Consequences of a Pressurized Water Reactor Radioactive Gas Storage Tank Failure (Safety Guide 24)	---	72/03
1.25	Assumptions Used for Evaluating the Potential Radiological Consequences of a Fuel Handling Accident in the Fuel Handling and Storage Facility for Boiling and Pressurized Water Reactors (Safety Guide 25)	---	72/03
1.26	Quality Group Classifications and Standards for Water-, Steam-, and Radioactive-Waste-Containing Components of Nuclear Power Plants (For Comment)	---	72/03
		1	74/09
		2	75/06
		3	76/02
1.27	Ultimate Heat Sink for Nuclear Power Plants (For Comment)	---	72/03
		1	74/03
		2	76/01
1.28	Quality Assurance Program Requirements (Design and Construction) (For Comment)	---	72/06
		1	78/03
		2	79/02

Number	Title	Rev.	Issued Year/Month
1.29	Seismic Design Classification	---	72/06
		1	73/08
		2	76/02
		3	78/09
1.30	Quality Assurance Requirements for the Installation, Inspection, and Testing of Instrumentation and Electric Equipment (Safety Guide 30)	---	72/08
1.31	Control of Ferrite Content in Stainless Steel Weld Metal	---	72/08
		1	73/06
		2	77/05
		3	78/04
1.32	Criteria for Safety-Related Electric Power Systems for Nuclear Power Plants	---	72/08
		1	76/03
		2	77/02
1.33	Quality Assurance Program Requirements (Operation)	---	72/11
		1	77/02
		2	78/02
1.34	Control of Electroslag Weld Properties	---	72/12
1.35	Inservice Inspection of UngROUTED Tendons in Prestressed Concrete Containment Structures	---	73/02
		1	74/06
		2	76/01
1.36	Nonmetallic Thermal Insulation for Austenitic Stainless Steel	---	73/02
1.37	Quality Assurance Requirements for Cleaning of Fluid Systems and Associated Components of Water-Cooled Nuclear Power Plants	---	73/03
1.38	Quality Assurance Requirements for Packaging, Shipping, Receiving, Storage, and Handling of Items for Water-Cooled Nuclear Power Plants	---	73/03
		1	76/10
		2	77/05
1.39	Housekeeping Requirements for Water-Cooled Nuclear Power Plants	---	73/03
		1	76/10
		2	77/09
1.40	Qualification Tests of Continuous-Duty Motors Installed Inside the Containment of Water-Cooled Nuclear Power Plants	---	73/03
1.41	Preoperational Testing of Redundant On-Site Electric Power Systems to Verify Proper Load Group Assignments	---	73/03
1.42	(Withdrawn--See 41 FR 11891, 3/22/76)	---	---
1.43	Control of Stainless Steel Weld Cladding of Low-Alloy Steel Components	---	73/05
1.44	Control of the Use of Sensitized Stainless Steel	---	73/05
1.45	Reactor Coolant Pressure Boundary Leakage Detection Systems	---	73/05
1.46	Protection Against Pipe Whip inside Containment	---	73/05
1.47	Bypassed and Inoperable Status Indication for Nuclear Power Plant Safety Systems	---	73/05
1.48	Design Limits and Loading Combinations for Seismic Category I Fluid System Components	---	73/05
1.49	Power Levels of Nuclear Power Plants	---	73/05
		1	73/12

Number	Title	Rev.	Issued Year/Month
1.50	Control of Preheat Temperature for Welding of Low-Alloy Steel	---	73/05
1.51	(Withdrawn--See 40 FR 30510, 7/21/75)	---	---
1.52	Design, Testing, and Maintenance Criteria for Post Accident Engineered-Safety-Feature Atmosphere Cleanup System Air Filtration and Adsorption Units of Light-Water-Cooled Nuclear Power Plants	---	73/06
		1	76/07
		2	78/03
1.53	Application of the Single-Failure Criterion to Nuclear Power Plant Protection Systems	---	73/06
1.54	Quality Assurance Requirements for Protective Coatings Applied to Water-Cooled Nuclear Power Plants	---	73/06
1.55	Concrete Placement in Category I Structures	---	73/06
1.56	Maintenance of Water Purity in Boiling Water Reactors (For Comment)	---	73/06
		1	78/07
1.57	Design Limits and Loading Combinations for Metal Primary Reactor Containment System Components	---	73/06
1.58	Qualification of Nuclear Power Plant Inspection, Examination, and Testing Personnel	---	73/08
1.59	Design Basis Floods for Nuclear Power Plants	---	73/08
		1	76/04
		2	77/08
1.60	Design Response Spectra for Seismic Design of Nuclear Power Plants	---	73/10
		1	73/12
1.61	Damping Values for Seismic Design of Nuclear Power Plants	---	73/10
1.62	Manual Initiation of Protective Actions	---	73/10
1.63	Electric Penetration Assemblies in Containment Structures for Light-Water-Cooled Nuclear Power Plants	---	73/10
		1	77/05
		2	78/07
1.64	Quality Assurance Requirements for the Design of Nuclear Power Plants	---	73/10
		1	75/02
		2	76/06
1.65	Materials and Inspections for Reactor Vessel Closure Studs	---	73/10
1.66	(Withdrawn--See 42 FR 54478, 10/06/77)	---	---
1.67	Installation of Overpressure Protection Devices	---	73/10
1.68	Initial Test Programs for Water-Cooled Reactor Power Plants	---	73/11
		1	77/01
		2	78/03
1.68.1	Preoperational and Initial Startup Testing of Feedwater and Condensate Systems for Boiling Water Reactor Power Plants	---	75/12
		1	77/01
1.68.2	Initial Startup Test Program to Demonstrate Remote Shutdown Capability for Water-Cooled Nuclear Power Plants	---	77/01
		1	78/07
1.69	Concrete Radiation Shields for Nuclear Power Plants	---	73/12

Number	Title	Rev.	Issued Year/Month
1.70	Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants	--- 1 2 3	72/02 72/10 75/09 78/11
3 1.71	Welder Qualification for Areas of Limited Accessibility	---	73/12
I 1.72	Spray Pond Piping Made from Fiberglass-Reinforced Thermosetting Resin	---	73/12 78/01 78/11
? 1.73	Qualification Tests of Electric Valve Operators Installed Inside the Containment of Nuclear Power Plants	---	74/01
— 1.74	Quality Assurance Terms and Definitions	---	74/02
CP I 1.75	Physical Independence of Electric Systems	---	74/02 75/01 78/09
? 1.76	Design Basis Tornado for Nuclear Power Plants	---	74/04
? 1.77	Assumptions Used for Evaluating a Control Rod Ejection Accident for Pressurized Water Reactors	---	74/05
? 1.78	Assumptions for Evaluating the Habitability of a Nuclear Power Plant Control Room During a Postulated Hazardous Chemical Release	---	74/06
Back Log I 1.79	Preoperational Testing of Emergency Core Cooling Systems for Pressurized Water Reactors	---	74/06 75/09
I 1.80	Preoperational Testing of Instrument Air Systems	---	74/06
I 1.81	Shared Emergency and Shutdown Electric Systems for Multi-Unit Nuclear Power Plants	---	74/06 75/01
I or 3? 1.82	Sumps for Emergency Core Cooling and Containment Spray Systems	---	74/06
3 1.83	Inservice Inspection of Pressurized Water Reactor Steam Generator Tubes	---	74/06 75/07
Ia3 1.84	Design and Fabrication Code Case Acceptability--ASME Section III, Division 1	---	74/06 75/04 75/06 75/09 75/11 76/02 76/05 76/08 76/11 77/03 77/08 77/11 78/03 78/07 78/11
I 1.85	Materials Code Case Acceptability--ASME Section III, Division 1	---	74/06 75/04 75/06 75/09 75/11 76/02 76/05

Number	Title	Rev.	Issued Year/Month
		7	76/08
		8	76/11
		9	77/03
		10	77/08
		11	77/11
		12	78/03
		13	78/07
		14	78/11
<i>I</i> 1.86	Termination of Operating Licenses for Nuclear Reactors	---	74/06
1.87	Guidance for Construction of Class 1 Components in Elevated-Temperature Reactors (Supplement to ASME Section III Code Cases 1592, 1593, 1594, 1595, and 1596)	---	74/06
		1	75/06
<i>I</i> 1.88	Collection, Storage, and Maintenance of Nuclear Power Plant Quality Assurance Records	---	74/08
		1	75/12
		2	76/10
<i>II</i> 1.89	Qualification of Class 1E Equipment for Nuclear Power Plants	---	74/11
<i>In II</i> 1.90	Inservice Inspection of Prestressed Concrete Containment Structures with Grouted Tendons	---	74/11
		1	77/08
<i>In III</i> 1.91	Evaluations of Explosions Postulated to Occur on Transportation Routes Near Nuclear Power Plants (For Comment)	---	75/01
		1	78/02
<i>In III</i> 1.92	Combining Modal Responses and Spatial Components in Seismic Response Analysis	---	74/12
		1	76/02
<i>I</i> 1.93	Availability of Electric Power Sources	---	74/12
<i>I</i> 1.94	Quality Assurance Requirements for Installation, Inspection, and Testing of Structural Concrete and Structural Steel During the Construction Phase of Nuclear Power Plants	---	75/04
		1	76/04
<i>I</i> 1.95	Protection of Nuclear Power Plant Control Room Operators Against an Accidental Chlorine Release	---	75/02
		1	77/01
<i>II</i> 1.96	Design of Main Steam Isolation Valve Leakage Control Systems for Boiling Water Reactor Nuclear Power Plants	---	75/05
		1	76/06
<i>I</i> 1.97	Instrumentation for Light-Water-Cooled Nuclear Power Plants To Assess Plant Conditions During and Following an Accident	---	75/12
		1	77/08
<i>I</i> 1.98	Assumptions Used for Evaluating the Potential Radiological Consequences of a Radioactive Offgas System Failure in a Boiling Water Reactor (For Comment)	---	76/03
<i>I or III</i> 1.99	Effects of Residual Elements on Predicted Radiation Damage to Reactor Vessel Materials	---	75/07
		1	77/04
<i>I</i> 1.100	Seismic Qualification of Electric Equipment for Nuclear Power Plants	---	76/03
		1	77/08
<i>I or III</i> 1.101	Emergency Planning for Nuclear Power Plants	---	75/11
		1	77/03
<i>I</i> 1.102	Flood Protection for Nuclear Power Plants	---	75/10
		1	76/09
<i>I</i> 1.103	Post-Tensioned Prestressing Systems for Concrete Reactor Vessels and Containments	---	75/11
		1	76/10

Number	Title	Rev.	Issued Year/Month
<i>III</i> 1.104	Overhead Crane Handling Systems for Nuclear Power Plants (For Comment)	---	76/02
<i>For II</i> 1.105	Instrument Setpoints	---	75/11
		1	76/11
<i>I</i> 1.106	Thermal Overload Protection for Electric Motors on Motor-Operated Valves	---	75/11
		1	77/03
<i>I</i> 1.107	Qualifications for Cement Grouting for Prestressing Tendons in Containment Structures	---	75/11
		1	77/02
<i>III</i> 1.108	Periodic Testing of Diesel Generator Units Used as Onsite Electric Power Systems at Nuclear Power Plants	---	76/08
		1	77/08
<i>I</i> 1.109	Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I.	---	76/03
		1	77/10
1.110	Cost-Benefit Analysis for Radwaste Systems for Light-Water-Cooled Nuclear Power Reactors (For Comment)	---	76/03
<i>I or ?</i> 1.111	Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases from Light-Water-Cooled Reactors	---	76/03
		1	77/07
<i>For ?</i> 1.112	Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Light-Water-Cooled Power Reactors	---	76/04
		O-R	77/05
<i>For III</i> 1.113	Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor Releases for the Purpose of Implementing Appendix I	---	76/05
		1	77/04
<i>For II</i> 1.114	Guidance on Being Operator at the Controls of a Nuclear Power Plant	---	76/02
		1	76/11
1.115	Protection Against Low-Trajectory Turbine Missiles	---	76/03
		1	77/07
<i>III</i> 1.116	Quality Assurance Requirements for Installation, Inspection, and Testing of Mechanical Equipment and Systems	---	76/06
		O-R	77/05
1.117	Tornado Design Classification	---	76/06
		1	78/04
<i>III</i> 1.118	Periodic Testing of Electric Power and Protection Systems	---	76/06
		1	77/11
		2	78/06
<i>I</i> 1.1	(Withdrawn--See 42 FR 33387, 6/30/77)	---	---
1.120	Fire Protection Guidelines for Nuclear Power Plants (For Comment)	---	76/06
		1	77/11
1.121	Bases for Plugging Degraded PWR Steam Generator Tubes (For Comment)	---	76/08
<i>I</i> 1.122	Development of Floor Design Response Spectra for Seismic Design of Floor-Supported Equipment or Components	---	76/09
		1	78/02
<i>III</i> 1.123	Quality Assurance Requirements for Control of Procurement of Items and Services for Nuclear Power Plants	---	76/10
		1	77/07

Number	Title	Rev.	Issued Year/Month
<i>III</i> 1.124	Service Limits and Loading Combinations for Class 1 Linear-Type Component Supports	--- 1	76/11 78/01
<i>For III</i> 1.125	Physical Models for Design and Operation of Hydraulic Structures and Systems for Nuclear Power Plants	--- 1	77/03 78/10
<i>III</i> 1.126	An Acceptable Model and Related Statistical Methods for the Analysis of Fuel Densification	--- 1	77/03 78/03
<i>III</i> 1.127	Inspection of Water-Control Structures Associated with Nuclear Power Plants	--- 1	77/04 78/03
<i>For III</i> 1.128	Installation Design and Installation of Large Lead Storage Batteries for Nuclear Power Plants	--- 1	77/04 78/10
<i>I</i> 1.129	Maintenance, Testing, and Replacement of Large Lead Storage Batteries for Nuclear Power Plants	--- 1	77/04 78/02
1.130	Service Limits and Loading Combinations for Class 1 Plate-and-Shell-Type Component Supports	--- 1	77/07 78/10
1.131	Qualification Tests of Electric Cables, Field Splices, and Connections for Light-Water-Cooled Nuclear Power Plants (For Comment)	---	77/08
1.132	Site Investigations for Foundations of Nuclear Power Plants	--- 1	77/09 79/03
1.133	Loose-Part Detection Program for the Primary System of Light-Water-Cooled Reactors (For Comment)	---	77/09
1.134	Medical Certification and Monitoring of Personnel Requiring Operator Licenses	--- 1	77/09 79/03
1.135	Normal Water Level and Discharge at Nuclear Power Plants (For Comment)	---	77/09
1.136	Material for Concrete Containments	--- 1	77/11 78/10
1.137	Fuel-Oil Systems for Standby Diesel Generators (For Comment)	---	78/01
1.138	Laboratory Investigations of Soils for Engineering Analysis and Design of Nuclear Power Plants (For Comment)	---	78/04
1.139	Guidance for Residual Heat Removal (For Comment)	---	78/05
1.140	Design, Testing, and Maintenance Criteria for Normal Ventilation Exhaust System Air Filtration and Adsorption Units of Light-Water-Cooled Nuclear Power Plants (For Comment)	---	78/03
1.141	Containment Isolation Provisions for Fluid Systems (For Comment)	---	78/04
1.142	Safety-Related Concrete Structures for Nuclear Power Plants (Other Than Reactor Vessels and Containments) (For Comment)	---	78/04
1.143	Design Guidance for Radioactive Waste Management Systems, Structures, and Components Installed in Light-Water-Cooled Nuclear Power Plants (For Comment)	---	78/07
1.144	Auditing of Quality Assurance Programs for Nuclear Power Plants (For Comment)	---	79/01

April 1979

DIVISION 1 - POWER REACTORS

DRAFT REGULATORY GUIDES

Task Number	Title	Issued Year/Month
EM 805-5	Nuclear Analysis and Design of Concrete Radiation Shielding for Nuclear Power Plants	79/02
RS 810-5	Qualification of Quality Assurance Program Audit Personnel for Nuclear Power Plants	79/02
SC 704-5	Functional Specification for Safety-Related Valve Assemblies in Nuclear Power Plants	79/02
SC 807-4 (Proposed R.G. 1.35.1)	Determining Prestressing Forces for Inspection of Prestressed Concrete Containments	79/04

April 1979

DIVISION 1 - POWER REACTORS
PROPOSED REVISIONS TO REGULATORY GUIDES

Task & R.G. Numbers	Title	Proposed Revision	Issued Year/Month
RS 807-5 1.8	Personnel Selection and Training	2	79/02
SC 810-4 1.35	Inservice Inspection of UngROUTED Tendons in Prestressed Concrete Containments	3	79/04

U.S. NUCLEAR REGULATORY COMMISSION
REGULATORY GUIDE SERIES

DIVISION 2 - RESEARCH AND TEST REACTORS

TABLE OF CONTENTS

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Division 2, one of ten broad divisions in which regulatory guides are issued, contains those guides that were developed for research and test reactors. There may also be some guides issued in other divisions that would be of interest to those whose primary concern is in the area of research and test reactors. Accordingly, this issue of the table of contents includes, for the first time, a listing of regulatory guides issued in the other divisions that the NRC staff has identified as possibly of interest to recipients of Division 2 guides. This listing will be updated from time to time, and suggestions for additions to it are encouraged.

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Regulatory Guides

Number	Title	Rev.	Issued Year/Month
2.1	Shield Test Program for Evaluation of Installed Biological Shielding in Research and Training Reactors	---	73/05
2.2	Development of Technical Specifications for Experiments in Research Reactors	---	73/11
2.3	Quality Verification for Plate-Type Uranium-Aluminum Fuel Elements for Use in Research Reactors	--- 1	75/09 76/07
2.4	Review of Experiments for Research Reactors	--- O-R	76/07 77/05

Number	Title	Rev.	Issued Year/Month
2.5	Quality Assurance Program Requirements for Research Reactors	--- O-R	77/05 77/10
2.6	Emergency Planning for Research Reactors (For Comment)	---	79/01

DIVISION 3 REGULATORY GUIDES
FUELS AND MATERIALS FACILITIES

TABLE OF CONTENTS

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Number	Title	Issued	
		Rev.	Year/Month
3.1	Use of Borosilicate-Glass Raschig Rings as a Neutron Absorber in Solutions of Fissile Material	---	73/01
3.2	Efficiency Testing of Air-Cleaning Systems Containing Devices for Removal of Particles	---	73/01
3.3	Quality Assurance Program Requirements for Fuel Reprocessing Plants and for Plutonium Processing and Fuel Fabrication Plants	---	73/01
		1	74/03
3.4	Nuclear Criticality Safety in Operations with Fissionable Materials Outside Reactors	---	73/01
		1	77/08
		1-R	78/02
3.5	Standard Format and Content of License Applications for Uranium Mills (For Comment)	---	73/02
		1	77/11
3.6	Content of Technical Specifications for Fuel Reprocessing Plants ²	---	73/04
3.7	Monitoring of Combustible Gases and Vapors in Plutonium Processing and Fuel Fabrication Plants	---	73/03
3.8	Preparation of Environmental Reports for Uranium Mills (For Comment)	---	73/04
		1	78/09

Number	Title	Rev.	Issued Year/Month
3.9	Concrete Radiation Shields	---	73/06
3.10	Liquid Waste Treatment System Design Guide for Plutonium Processing and Fuel Fabrication Plants	---	73/06
3.11	Design, Construction, and Inspection of Embankment Retention Systems for Uranium Mills	---	73/06 1 77/03 2 77/12
3.11.1	Operational Inspection and Surveillance of Embankment Retention Systems for Uranium Mill Tailings (For Comment)	---	79/04
3.12	General Design Guide for Ventilation Systems of Plutonium Processing and Fuel Fabrication Plants	---	73/08
3.13	Guide for Acceptable Waste Storage Methods at UF ₆ Production Plants	---	73/10
3.14	Seismic Design Classification for Plutonium Processing and Fuel Fabrication Plants	---	73/10
3.15	Standard Format and Content of License Applications for Storage Only of Unirradiated Reactor Fuel and Associated Radioactive Material	---	73/10
3.16	General Fire Protection Guide for Plutonium Processing and Fuel Fabrication Plants	---	74/01
3.17	Earthquake Instrumentation for Fuel Reprocessing Plants	---	74/02
3.18	Confinement Barriers and Systems for Fuel Reprocessing Plants	---	74/02
3.19	Reporting of Operating Information for Fuel Reprocessing Plants	---	74/02
3.20	Process Offgas Systems for Fuel Reprocessing Plants	---	74/02
3.21	Quality Assurance Requirements for Protective Coatings Applied to Fuel Reprocessing and to Plutonium Processing and Fuel Fabrication Plants	---	74/03
3.22	Periodic Testing of Fuel Reprocessing Plant Protection System Actuation Functions	---	74/06
3.23	Stabilization of Uranium-Thorium Milling Waste Retention Systems	---	74/11
3.24	Guidance on the License Application, Siting, Design, and Plant Protection for an Independent Spent Fuel Storage Installation	---	74/12

Number	Title	Rev.	Issued Year/Month
3.25	Standard Format and Content of Safety Analysis Reports for Uranium Enrichment Facilities	---	74/12
3.26	Standard Format and Content of Safety Analysis Reports for Fuel Reprocessing Plants	---	75/02
3.27	Nondestructive Examination of Welds in the Liners of Concrete Barriers in Fuel Reprocessing Plants	--- 1	75/05 77/05
3.28	Welder Qualification for Welding in Areas of Limited Accessibility in Fuel Reprocessing Plants and in Plutonium Processing and Fuel Fabrication Plants	---	75/05
3.29	Preheat and Interpass Temperature Control for the Welding of Low-Alloy Steel for Use in Fuel Reprocessing Plants and in Plutonium Processing and Fuel Fabrication Plants	---	75/05
3.30	Selection, Application, and Inspection of Protective Coatings (Paints) for Fuel Reprocessing Plants	--- O-R	75/06 77/05
3.31	Emergency Water Supply Systems for Fuel Reprocessing Plants	--- O-R	75/09 77/05
3.32	General Design Guide for Ventilation Systems for Fuel Reprocessing Systems (For Comment)	---	75/09
3.33	Assumptions Used for Evaluating the Potential Radiological Consequences of Accidental Nuclear Criticality in a Fuel Reprocessing Plant (For Comment)	---	77/04
3.34	Assumptions Used for Evaluating the Potential Radiological Consequences of Accidental Nuclear Criticality in a Uranium Fuel Fabrication Plant (For Comment)	---	77/04
3.35	Assumptions Used for Evaluating the Potential Radiological Consequences of Accidental Nuclear Criticality in a Plutonium Processing and Fuel Fabrication Plant (For Comment)	---	77/05
3.36	(Withdrawn -- See 44 FR 6535 02/01/79)	---	---
3.37	Guidance for Avoiding Intergranular Corrosion and Stress Corrosion in Austenitic Stainless Steel Components of Fuel Reprocessing Plants (For Comment)	---	75/09
3.38	General Fire Protection Guide for Fuel Reprocessing Plants (For Comment)	---	76/06
3.39	Standard Format and Content of License Applications for Plutonium Processing and Fuel Fabrication Plants	---	76/01
3.40	Design Basis Floods for Fuel Reprocessing Plants and for Plutonium Processing and Fuel Fabrication Plants	--- 1	76/11 77/12

Number	Title	Issued	
		Rev.	Year/Month
3.41	Validation of Calculational Methods for Nuclear Criticality Safety	---	76/06
		1	77/05
3.42	Emergency Planning for Fuel Cycle Facilities and Plants Licensed Under 10 CFR Parts 50 and 70 (For Comment)	---	77/08
3.43	Nuclear Criticality Safety in the Storage of Fissile Materials	---	78/08
		1	79/04
3.44	Standard Format and Content for the Safety Analysis Report To Be Included in a License Application for the Storage of Spent Fuel in an Independent Spent Fuel Storage Installation (Water-Basin Type)	---	78/12

Draft Regulatory Guides

Task Number	Title	Issued Year/Month
RH 802-4	Calculational Models for Estimating Radiation Doses to Man From Airborne Radioactive Materials Resulting From Uranium Milling Operations	79/05

DIVISION 4 REGULATORY GUIDES
ENVIRONMENTAL AND SITING

TABLE OF CONTENTS

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Number	Title	Rev.	Issued Year/Month
4.1	Programs for Monitoring Radioactivity in the Environs of Nuclear Power Plants (For Comment)	--- 1	73/01 75/04
4.2	Preparation of Environmental Reports for Nuclear Power Stations	--- 1 2	71/02 75/01 76/07
4.3	(WITHDRAWN--See 41 FR 53870, 12/09/76)		
4.4	Reporting Procedure for Mathematical Models Selected to Predict Heated Effluent Dispersion in Natural Water Bodies	---	74/05
4.5	Measurements of Radionuclides in the Environment--Sampling and Analysis of Plutonium in Soil	---	74/05
4.6	Measurements of Radionuclides in the Environment---Strontium-89 and Strontium-90 Analyses	---	74/05
4.7	General Site Suitability Criteria for Nuclear Power Stations	--- 1	74/09 75/11
4.8	Environmental Technical Specifications for Nuclear Power Plants (For Comment)	---	75/12
4.9	Preparation of Environmental Reports for Commercial Uranium Enrichment Facilities	--- 1	74/12 75/10

Number	Title	Rev.	Issued Year/Month
4.10	(WITHDRAWN--See 42 FR 59436, 11/17/77)		
4.11	Terrestrial Environmental Studies for Nuclear Power Stations	--- 1	76/07 77/08
4.12	(Not yet published)		
4.13	Performance, Testing, and Procedural Specifications for Thermoluminescence Dosimetry: Environmental Applications	--- 1	76/11 77/07
4.14	Measuring, Evaluating, and Reporting Radioactivity in Releases of Radioactive Materials in Liquid and Airborne Effluents from Uranium Mills (For Comment)	---	77/06
4.15	Quality Assurance for Radiological Monitoring Programs (Normal Operations)--Effluent Streams and the Environment	--- 1	77/12 79/02
4.16	Measuring, Evaluating, and Reporting Radioactivity in Releases of Radioactive Materials in Liquid and Airborne Effluents from Nuclear Fuel Processing and Fabrication Plants (For Comment)	---	78/03

Proposed Revisions to Regulatory Guides

R.G. and Task Numbers	Title	Proposed Revision	Issued Year/Month
5.7 (SG 909-4)	Entry/Exit Control to Protected Areas, Vital Areas, and Material Access Areas	1	79/05
5.14 (SG 910-4)	Use of Observation (Visual Surveillance) Techniques in Material Access Areas	1	79/05
5.44 (SG 479-4)	Perimeter Intrusion Alarm Systems	2	79/05
5.57 (SG 908-4)	Shipping and Receiving Control of Strategic Special Nuclear Material	1	79/05

DIVISION 5 REGULATORY GUIDES
MATERIALS AND PLANT PROTECTION

TABLE OF CONTENTS

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Number	Title	Rev.	Issued Year/Month
5.1	Serial Numbering of Fuel Assemblies for Light-Water-Cooled Nuclear Power Reactors	---	72/12
5.2	Classification of Unirradiated Plutonium and Uranium Scrap	---	72/12
5.3	Statistical Terminology and Notation for Special Nuclear Materials Control and Accountability	---	73/02
5.4	Standard Analytical Methods for the Measurement of Uranium Tetrafluoride (UF ₄) and Uranium Hexafluoride (UF ₆)	---	73/02
5.5	Standard Methods for Chemical, Mass Spectrometric, and Spectrochemical Analysis of Nuclear-Grade Uranium Dioxide Powders and Pellets	---	73/02
5.6	Standard Methods for Chemical, Mass Spectrometric, and Spectrochemical Analysis of Nuclear-Grade Plutonium Dioxide Powders and Pellets and Nuclear-Grade Mixed Oxides ([U,Pu]O ₂)	---	73/05
5.7	Control of Personnel Access to Protected Areas, Vital Areas, and Material Access Areas	---	73/06
5.8	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material in Drying and Fluidized Bed Operations	---	73/06 1 74/05
5.9	Specifications of Ge(Li) Spectroscopy Systems for Material Protection Measurements--Part I: Data Acquisition Systems	---	73/06 1 74/05

Number	Title	Rev.	Issued Year/Month
5.10	Selection and Use of Pressure-Sensitive Seals on Containers for On-site Storage of Special Nuclear Material	---	73/07
5.11	Nondestructive Assay of Special Nuclear Material Contained in Scrap and Waste	---	73/10
5.12	General Use of Locks in the Protection and Control of Facilities and Special Nuclear Materials	---	73/11
5.13	Conduct of Nuclear Material Physical Inventories	---	73/11
5.14	Visual Surveillance of Individuals in Material Access Areas	---	73/11
5.15	Security Seals for the Protection and Control of Special Nuclear Material	---	74/01
5.16	Standard Methods for Chemical, Mass Spectrometric, Spectrochemical, Nuclear, and Radiochemical Analysis of Nuclear-Grade Plutonium Nitrate Solutions and Plutonium Metal (For Comment)	--- 1	74/01 75/05
5.17	Truck Identification Markings	---	74/01
5.18	Limit of Error Concepts and Principles of Calculation in Nuclear Materials Control	---	74/01
5.19	Methods for the Accountability of Plutonium Nitrate Solutions	---	74/01
5.20	Training, Equipping, and Qualifying of Guards and Watchmen	---	74/01
5.21	Nondestructive Uranium-235 Enrichment Assay by Gamma-Ray Spectrometry	---	74/04
5.22	Assessment of the Assumption of Normality (Employing Individual Observed Values)	---	74/04
5.23	In Situ Assay of Plutonium Residual Holdup	---	74/05
5.24	Analysis and Use of Process Data for the Protection of Special Nuclear Material	---	74/06
5.25	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material in Equipment for Wet Process Operations	---	74/06
5.26	Selection of Material Balance Areas and Item Control Areas	--- 1	74/06 75/04
5.27	Special Nuclear Material Doorway Monitors	---	74/06
5.28	Evaluation of Shipper-Receiver Differences in the Transfer of Special Nuclear Materials	---	74/06
5.29	Nuclear Material Control Systems for Nuclear Power Plants	--- 1	74/06 75/06
5.30	Materials Protection Contingency Measures for Uranium and Plutonium Fuel Manufacturing Plants	---	74/06
5.31	Specially Designed Vehicle With Armed Guards for Road Shipment of Special Nuclear Material	--- 1	74/06 75/04
5.32	Communication With Transport Vehicles	--- 1	74/06 75/05
5.33	Statistical Evaluation of Material Unaccounted For	---	74/06

Number	Title	Rev.	Issued Year/Month
5.34	Nondestructive Assay for Plutonium in Scrap Material by Spontaneous Fission Detection	---	74/06
5.35	(Withdrawn -- See 42 FR 41677, 8/18/77)	---	---
5.36	Recommended Practice for Dealing With Outlying Observations	---	74/06
5.37	In Situ Assay of Enriched Uranium Residual Holdup	---	74/08
5.38	Nondestructive Assay of High-Enrichment Uranium Fuel Plates by Gamma-Ray Spectrometry	---	74/09
5.39	General Methods for the Analysis of Uranyl Nitrate Solutions for Assay, Isotopic Distribution, and Impurity Determinations	---	74/12
5.40	Methods for the Accountability of Plutonium Dioxide Powder	---	74/12
5.41	(Not issued)	---	---
5.42	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material in Equipment for Dry Process Operation	---	75/01
5.43	Plant Security Force Duties	---	75/01
5.44	Perimeter Intrusion Alarm Systems	---	75/01 1 76/06
5.45	Standard Format and Content for the Special Nuclear Material Control and Accounting Section of a Special Nuclear Material License Application (Including That for a Uranium Enrichment Facility)	---	74/12
5.46	(Not issued)	---	---
5.47	Control and Accountability of Plutonium in Waste Material	---	75/02
5.48	Design Considerations--Systems for Measuring the Mass of Liquids	---	75/02
5.49	Internal Transfers of Special Nuclear Material (For Comment)	---	75/03
5.50	(Not issued)	---	---
5.51	Management Review of Nuclear Material Control and Accounting Systems (For Comment)	---	75/06
5.52	Standard Format and Content for the Physical Protection Section of a License Application (for Facilities Other Than Nuclear Power Plants)(For Comment)	---	75/05 1 76/06
	Revisions (For 60-Day Comment) to Chapter 4, "Security Organization," and Chapter 18, "Security Personnel," of Regulatory Guide 5.52, Revision 1 (For Comment) were issued July 1978.		
5.53	Qualification, Calibration, and Error Estimation Methods for Non-destructive Assay (For Comment)	---	75/08
5.54	Standard Format and Content of Safeguards Contingency Plans for Nuclear Power Plants (For Comment)	---	78/03
5.55	Standard Format and Content of Safeguards Contingency Plans for Fuel Cycle Facilities (For Comment)	---	78/03
5.56	Standard Format and Content of Safeguards Contingency Plans for Transportation (For Comment)	---	78/03

Number	Title	Rev.	Issued Year/Month
5.57	Shipping and Receiving Control of Special Nuclear Material (For Comment)	---	76/06
5.58	Considerations for Establishing Traceability of SNM Accounting Measurements (For Comment)	---	78/11

U. S. NUCLEAR REGULATORY COMMISSION
REGULATORY GUIDE SERIES

DIVISION 6 - PRODUCTS

TABLE OF CONTENTS

Previous issues of this table of contents listed only the latest revision of each regulatory guide in Division 6 because it represented a current NRC staff position. However, some licensing commitments were based on earlier versions of a guide; therefore, this and future issues of the table of contents will list every version of each guide in Division 6 with the date it was issued. If the latest version of a guide (as of the date of this table of contents) was issued for early comment, this is so noted.

Division 6, one of ten broad divisions in which regulatory guides are issued, contains those guides that were developed to provide guidance in the area of products. There may also be some guides issued in other divisions that would be of interest to those whose primary concern is in the area of products. Accordingly, this issue of the table of contents includes, for the first time, a listing of regulatory guides issued in the other divisions that the NRC staff has identified as possibly of interest to recipients of Division 6 guides. This listing will be updated from time to time, and suggestions for additions to it are encouraged.

Most regulatory guides contain a section headed "Implementation" that is intended to provide information to applicants and licensees regarding the NRC staff's plans for using the guide. If a guide does not contain such a section or if detailed information is needed on the staff's plans for using a regulatory guide with respect to a specific permit or license or application therefor, requests for such information should be addressed to the appropriate licensing project manager in the Office of Nuclear Reactor Regulation or Office of Nuclear Material Safety and Safeguards.

At an appropriate point in the development of a new regulatory guide or a proposed revision to an existing guide, the guide and the associated value/impact statement are issued in draft form to involve the public in the early stages of the development of a regulatory position. These drafts have not received complete staff review and do not represent an official NRC staff position. They are temporarily identified by their task number and issued to the same distribution list that is used for published guides in each division. Lists of these drafts will be included in future issues of the table of contents.

All regulatory guides, including draft guides, proposed revisions, and all published revisions, may be examined at the Commission's Public Document Room at 1717 H Street NW., Washington, D.C. Requests for single copies of draft guides, proposed revisions, and the latest revision of published guides, the only version currently in print, should be made in writing to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Technical Information and Document Control. Regulatory guides are not copyrighted and Commission approval is not required to reproduce them.

Regulatory Guides

Number	Title	Rev.	Issued Year/Month
6.1	Leak Testing Radioactive Brachytherapy Sources	---	74/02
		1	74/07
6.2	Integrity and Test Specifications for Selected Brachytherapy Sources	---	74/02
		1	74/07
6.3	Design, Construction, and Use of Radioisotopic Power Generators for Certain Land and Sea Applications	---	74/03

Number	Title	Rev.	Issued Year/Month
6.4	Classification of Containment Properties of Radioactive Sources Contained in Certain Devices To Be Distributed for Use Under General License	--- 1	74/03 75/05
6.5	General Safety Standard for Installations Using Nonmedical Sealed Gamma-Ray Sources	---	74/06
6.6	Acceptance Sampling Procedures for Exempted and Generally Licensed Items Containing Byproduct Material	---	74/06
6.7	Preparation of an Environmental Report To Support a Rule Making Petition Seeking an Exemption for a Radionuclide-Containing Product	--- 1	75/10 76/06
6.8	Identification Plaque for Irretrievable Well-Logging Sources (For Comment)	---	78/10

March 1979

DIVISION 7 - TRANSPORTATION
TABLE OF CONTENTS
Regulatory Guides

Number	Title	Rev.	Issued Year/Month
7.1	Administrative Guide for Packaging and Transporting Radioactive Material	---	74/06
7.2	Packaging and Transportation of Radioactively Contaminated Biological Materials	---	74/06
7.3	Procedures for Picking Up and Receiving Packages of Radioactive Material (For Comment)	---	75/05
7.4	Leakage Tests on Packages for Shipment of Radioactive Materials (For Comment)	---	75/06
7.5	Administrative Guide for Obtaining Exemptions From Certain NRC Requirements Over Radioactive Material Shipments	--- 0-R	75/06 77/05
7.6	Design Criteria for the Structural Analysis of Shipping Cask Containment Vessels	--- 1	77/02 78/03
7.7	Administrative Guide for Verifying Compliance With Packaging Requirements for Shipments of Radioactive Materials (For Comment)	---	77/08
7.8	Load Combinations for the Structural Analysis of Shipping Casks	---	77/05
7.9	Standard Format and Content of Part 71 Applications for Approval of Packaging of Type B, Large Quantity, and Fissile Radioactive Material (For Comment)	---	79/03



U.S. NUCLEAR REGULATORY COMMISSION

REGULATORY GUIDE

OFFICE OF STANDARDS DEVELOPMENT

DIVISION 8 REGULATORY GUIDES
OCCUPATIONAL HEALTH

TABLE OF CONTENTS

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Number	Title	Rev.	Issued
8.1	Radiation Symbol	—	2/2/73
8.2	Guide for Administrative Practices in Radiation Monitoring	—	2/2/73
8.3	Film Badge Performance Criteria	—	2/2/73
8.4	Direct-Reading and Indirect-Reading Pocket Dosimeters	—	2/26/73
8.5	Immediate Evacuation Signal	—	2/16/73
8.6	Standard Test Procedure for Geiger-Müller Counters	—	5/73
8.7	Occupational Radiation Exposure Records Systems	—	5/73
8.8	Information Relevant to Ensuring That Occupational Radiation Exposures at Nuclear Power Stations Will Be As Low As Is Reasonably Achievable	— 1 2 3	7/73 9/75 3/77 6/78
8.9	Acceptable Concepts, Models, Equations, and Assumptions for a Bioassay Program	—	9/73
8.10	Operating Philosophy for Maintaining Occupational Radiation Exposures As Low As Is Reasonably Achievable	— 1 1-R	4/74 9/75 5/77
8.11	Applications of Bioassay for Uranium	—	6/74
8.12	Criticality Accident Alarm Systems	—	12/74
8.13	Instruction Concerning Prenatal Radiation Exposure	— 1	3/75 11/75

Number	Title	Rev.	Issued
8.14	Personnel Neutron Dosimeters	— 1	6/76 8/77
8.15	Acceptable Programs for Respiratory Protection	—	10/76
8.16	(Not yet published)		
8.17	(Not yet published)		
8.18	Information Relevant to Ensuring That Occupational Radiation Exposures at Medical Institutions Will Be As Low As Reasonably Achievable (For Comment)	—	12/77
8.19	Occupational Radiation Dose Assessment in Light-Water Reactor Power Plants—Design Stage Man-Rem Estimates (For Comment)	—	5/78
8.20	Applications of Bioassay for I-125 and I-131 (For Comment)	—	4/78
8.21	Health Physics Surveys for Byproduct Material at NRC-Licensed Processing and Manufacturing Plants (For Comment)	—	5/78
8.22	Bioassay at Uranium Mills (For Comment)	—	7/78
8.23	Radiation Safety Surveys at Medical Institutions (For Comment)	--	2/79
8.24	Health Physics Surveys During Enriched Uranium-235 Processing and Fuel Fabrication (For Comment)	--	11/78

April 1979

DIVISION 8 - OCCUPATIONAL HEALTH
PROPOSED REVISIONS TO REGULATORY GUIDES

Task and R.G. Numbers	Title	Proposed Revision	Issued Year/Month
OH 507-4 8.8	Information Relevant to Ensuring That Occupational Radiation Exposures at Nuclear Power Stations Will Be As Low As Is Reasonably Achievable (ALARA)	4	79/03

October 1978

DIVISION 9 REGULATORY GUIDES
ANTITRUST AND FINANCIAL REVIEW

TABLE OF CONTENTS

Number	Title
9.1	Regulatory Staff Position Statement on Antitrust Matters (12/73)
9.2	Information Needed by the NRC Staff in Connection with Its Antitrust Review of Construction Permit Applications for Nuclear Power Plants (Revision 1, 6/76)
9.3	Information Needed by the AEC Regulatory Staff in Connection with Its Antitrust Review of Operating License Applications for Nuclear Power Plants (10/74)
9.4	Suggested Format for Cash Flow Statements Submitted as Guarantees of Payment of Retrospective Premiums (9/78)

January 1979

DIVISION 10 REGULATORY GUIDES
GENERAL

TABLE OF CONTENTS

Number	Title
10.1	Compilation of Reporting Requirements for Persons Subject to NRC Regulations (Revision 3, 5/77)
10.2	Guidance to Academic Institutions Applying for Specific Byproduct Material Licenses (Revision 1, 12/76)
10.3	Guide for the Preparation of Applications for Special Nuclear Material Licenses of Less Than Critical Mass Quantities (Rev. 1, 5/77)
10.4	Guide for the Preparation of Applications for Licenses to Process Source Material (Revision 1, 3/77)
10.5	Guide for the Preparation of Applications for Type A Licenses of Broad Scope for Byproduct Material (9/76)
10.6	Guide for the Preparation of Applications for Use of Sealed Sources and Devices for the Performance of Industrial Radiography (9/76)
10.7	Guide for the Preparation of Applications for Licenses for Laboratory Use of Small Quantities of Byproduct Material (2/77)
10.8	Guide for the Preparation of Applications for Medical Programs (For Comment) (1/79)

March 1979

DIVISION 10 - GENERAL
Draft Regulatory Guides

Task Number	Title	Issued Year/Month
OH 706-4	Guide for Preparation of Applications for the Use of Gamma Irradiators	79/02