

CONTINUATION OF 00100759

FACILITY

CATEGORY

SOURCE

DOC ID

REL DATE

REPORT DATE

PWR 3

ALL SYSTEMS

05000001

100314

10/10/74

DESCRIPTION OF EVENT: During a surveillance test, a KACK POSITION VALVE FOR DRYER LINE PRESSURE SWITCH PS 3-5-11C WAS FOUND CLOSED. OPENING THE VALVE, THE VALVE WAS NOT OPENED FOLLOWING PRESS TESTING. EVEN THOUGH MEASURES WERE SPECIFIC OPERATOR ERRED IN CHECKING THE VALVE STATUS NO REMAINANT DEVICE WAS OPERABLE. (AO-74-10)

STATCH NO.

PROXIMATE
CAUSE CODE

A-PERSONNEL ERROR
B-DESIGN ERROR
C-EXTERNAL CAUSE
D-OTHER

COMPOSITE
CODE

10

4

3,4

DESCRIPTION OF CAUSE:

OPERATIVE ERROR. A PREVIOUS OCCURRENCE WAS REPORTED ON AO-50-277-74-36.

STATUS OF
REACTOR CORE

A-INITIAL
B-POWER TRANSIENT, INITIAL
STARTUP AND POWER ASCENSION
TESTS
C-ROUTINE STARTUP OPERATION
D-ROUTINE SHUTTING DOWN OPERATION
E-STADY STATE OPERATION AT
POWER

F-ROUTINE CHANGING OF
ROUTINE POWER OPERATION
G-SHUTDOWN (HOT OR COLD),
EXCEPT REFUELING
H-REFUELING
I-OTHER, INCLUDING SPECIAL
TESTS (DESCRIBE)

METHOD
OF
DISCOVERY
(DISC.):

A-COMMON MODE
B-ROUTINE TEST
C-SPECIAL TEST/ATTEMPT
D-EXTERNAL SOURCE

STATUS

X-POWER

OTHER STATUS:

DISC.

DESCRIPTION:

FORM OF
ACTIVITY
RELEASED

L-LIQUID
S-SOLID
G-GAS

CONTINUE
OF
RELEASE

N-EARLY GAS
H-HALOGEN
P-PARTICULATE

AMOUNT OF ACTIVITY:

LOCATION OF RELEASE:

PERSONNEL EXPOSURES

NUMBER: DESCRIPTION:

PERSONNEL INJURIES

NUMBER: DESCRIPTION:

OPPOSITE CONSEQUENCES:

DAMAGE TO FACILITY:

PUBLICITY:

ADDITIONAL FACTORS:

8002 100043

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PEACH BOTTOM ATOMIC POWER COMPANY

1-1000-11-1000

1000-1000-1000

1000-1000-1000

1000-1000-1000

1000-1000-1000

Mr. A. Glensbury
Deputy Director of Nuclear Programs
United States Atomic Energy Commission
Directorate of Licensing
Washington, D.C. 20449

Date: May 1975

Subject: Abnormal Occurrence

The following occurrence was reported to Mr. Paul Baunack, Associate Region II Regulatory Operations Officer on October 4, 1974. Verification notification was made to Mr. James P. O'Reilly, Region II Regulatory Operations Officer on October 4, 1974. In accordance with Section 6.7.2.A of the Technical Specifications, Appendix A of DPR-56 for Unit 3 Peach Bottom Atomic Power Station, the following report is being submitted to the Directorate of Licensing as an Abnormal Occurrences.

Reference: License Number DPR-56

Technical Specification Reference: Table 3-1-1

Report No.:	50-278-74-10
Report Date:	October 13, 1974
Occurrence Date:	October 3, 1974
Facility:	Peach Bottom Atomic Power Station R.D. 1, Delta, Pennsylvania

Identification of Occurrence:

Improper valving of one drywell high pressure scram switch.

Conditions Prior to Occurrence:

The reactor had been shutdown 16 days for a maintenance outage.

Description of Occurrence:

During a routine instrument surveillance test, the local isolation valve for drywell high pressure scram switch PS-3-5-120 was found to be in the shut position. Isolation of the device from its sensing line renders it inoperable.

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After shutdown, the reactor was brought back up to full power. No explosive hazard was detected. Determination of the cause has not been completed to determine whether or not the detector had been operated. The shutdown protection system was performed and the protection system was checked connected and procedures reviewed and were found to be correct. No positive indication was determined. It was believed, however, that in operation the system after its most recent testing, due to the logic techniques, may have checked and verified a neighboring valve by mistake, thereby resulting to properly. At two sites, instruments into service.

Anomalous Occurrence

This device is connected in a bridge circuit with logic configurations. Unoperability of one device does not prevent the protective system from performing its intended function. The other device's monitoring arms in pressure were checked and their valves found in the proper positions. For the above reasons, this occurrence is of minimal safety significance.

Cause Unknown

The other occurrence of this type will previously reported at the General Occurrence Summary.

Very truly yours,

M. J. Cooney
Area Gen. Superintendent
Generation Division

cc: Mr. J. P. O'Reilly
Director, Region I
United States Atomic Energy Commission
631 Park Avenue
King of Prussia PA 19406