

## Transcript of Proceedings

NUCLEAR REGULATORY COMMISSION

TMI SPECIAL INQUIRY GROUP

## INTERVIEW OF WILLIAM CAVANAUGH PART I

(THIS TRANSCRIPT WAS PREPARED FROM A TAPE RECORDING)

PLACE:

Little Rock, Arkansas

DATE:

Tuesday, 27 November 1979

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Little Rock, Arkansas Tuesday, 27 November 1979

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RCmte	1	Herr Vo S: Okay, we can get started.
	2	Today is November 27, 979. It is 9:06, and we
	3	are in the conference room on the eighth floor of the
di terre		
	4	Arkansas Power & Light offices in Little Rock, Arkansas.
	5	Present for this interview are Mr. William
	6	Cavanaugh, Vice President, Generation and Construction,
	7	Arkansas Power & Light, and Stephen Riggs, of House, Holmes,
	8	and Jewell, representing Arkansas Power & Light. Present
	Ŷ	from the Special Inquiry Group are James Creswell and
	10	Frederick Hurr.
	11	Mr. Cavanaugh, we have given you a copy of a
	12	document entitled "NRC Special Inquiry Group Witness
	13	Notification." Have you read the document?
	14	Cavanaugh VOICE: Yes, I have.
•	15	Herr VOICE: Do you understand the information
	16	contained in it?
	17	Cavanaugh VOICE: Yes.
	18	Herr VOICE: Okay. Mr. Cavanaugh, we could start with
	19	maybe you could give us a brief history of your
	20	nuclear-related ejucation and experience up to the present
	21	time.
	22	Cavanaugh VOICE: Well, let's see. Related to nuclear, my
	23	experience goes back to 1962, I believe it was, when I was
	24	selected to while in the Navy as a naval officer, I was
)	25	selected to go into the Naval Nuclear Power Program.

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RCmte	1	Basically, from that point until 1969 I was in the naval
	2	nuclear program, qualified as a chief engineer on the S5W
	3	naval nuclear plant. And 1969 to the present I have been
)	4	with I have been with Arkansas Power & Light and involved
	5	with its nuclear program in various capacities.
	0	Creswell VOICE: This is Jim Creswell speaking.
	7	Mr. Cavanaugh, I believe you told us that you are
	8	presently Vice President, Generation and Construction. How
	¥	long have you served in that capacity?
	10	Cavanaugh VOICE: Since January of this year.
	11	Creswell VOICE: Jim Creswell again.
	12	What are your responsibilities in that position?
	13	Cavanaugh VOICE: Well, my responsibilities in generation
	14	and construction are basically over six departments. We
•	15	have a generation operations department which has in it both
	16	the fossil and the nuclear operations, as well as the
	17	maintenance.
	18	The next department is the project management
	19	department, which has responsibility at the present time for
	20	three of our fossil projects.
	21	The next department is the generation engineering,
	22	which has the various disciplines in it responsible for
	23	providing the various engineering support for both the
	24	operating plants as well as the projects.
	25	We have a (Inau ible) environmental services

NRCmte

department, which has in it quality assurance, nuclear fuel
 management, licensing, and technical analysis, which is
 basically chemistry and the environmental support.

A generation technology department, that has in it availability engineering, plant performance evaluation, and generation research and development.

7 The last department is administrative services and 8 project support, which provides contract administration, 9 administrative services, planning and scheduling, cost 10 control and training, generation training.

II Creswell VOICE: Jim Creswell again.

In your present position, how do you interface
with the vendor for your Arkansas Nuclear Plant Unit 1, B&W,
Babcock & Wilcox?

15 Cavanaugh VOICE: Well, I am not the key interface with 16 Babcock & Wilcox. We have interfaces with B&W through our 17 licensing section, through our nuclear operations section, 18 of course Arkansas Nuclear I itself. And there may be an 19 occasional interface -- well, the fuel management section 20 would have interface. And then there may be occasional 21 interface with the other groups.

22 Creswell VOICE: Mr. Cavanaugh -- Jim Creswell speaking --23 who do you report to in the Arkansas Power & Light 24 organization?

25 Cavanaugh VOICE: I report to the president and chief

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executive officer, Mr. Gerry Malden. NRCmte 1 VOICE: I see. Now, I believe you said in January 2 Creswell. 1978 you started serving in your present capacity. 3 VOICE: January '79. Cavanaugh 4 Creswell VOICE: January '79. Excuse me. 5 Prior to that time, what was your position with 6 7 the company? Cavanaugh VOICE: Prior to that time, I was executive 8 director of generation construction, having been appointed 9 to that position in August of '77. 10 VOICE: In the time period of approximately August 11 Creswell of 1974 until May of 1975, what would your position have 12 been with Arkansas Power & Light? 13 Cavanaugh VOICE: Manager of nuclear services, primarily --14 15 at that time, the organization that was set up to support the operating plant, as well as continue the support and 16 17 management of the Arkansas Unit 2, Arkansas Nuclear 1, Unit 18 2 Project. 19 Creswell VOICE: Okay. What were your primary responsibilities in your position at that time as manager of 20 21 nuclear operations? Cavanaugh VOICE: Well, at that time I had the 22 responsibility for the technical support of Arkansas Nuclear 23 24 1. Unit I and 2. I had under me a licensing group, fuel management group. In the project group, we had the 25

8630 01 05 6 mechanical and electrical groups that were supporting NRCmte 1 primarily Unit 1 and Unit 2. Let's see. 2 3 I believe that those were the key elements of that 4 organization at that time. VOICE: What responsibilities did you have for 5 Creswell 6 interfacing with Babcock & Wilcox Company at that point in 7 time? VOICE: Well, at that time I was -- I was one of 8 Cavanaugh the key interfaces with B&W on items related to the support 4 -- at that time, let's see, '74? Unit I was still in 10 startup testing, and so as a result I was a key interface 11 12 with B&W on the project that was still in effect at that 13 time. And of course, Unit I went commercial December '74, 14 and we then continued -- I continued my involvement as an interface with B&W through -- you said May of '75? 15 Creswell 16 VOICE: Yes. 17 VOICE: Through that time. Where I was the key Cavanaugh 18 contact. Of course, there were other contacts with B&W from 19 our company. Creswell 20 VOICE: Would it be a fair characterization that, 21 regarding technical issues that developed during startup 22 testing of Unit I, that you would have been the primary 23 interface with Babcock & Wilcox Company? Cavanaugh VOICE: No, not necessarily. B&W had on site a 24 25 startup support group, and really the key interface

NRCmte 1 related to the test program was between the plant staff and 2 the B&W support group on site. creswell. VOICE: Was that centralized in any one 3 individual. that contact with B&W? 4 Cavanaugh 5 VOICE: AP&L's contact? Creswell 6 VOICE: Yes. VOICE: Well, of course, the plant -- I guess we 7 Cavanaugh would call him the plant superintendent at that time would 8 have been the contact. But of course, he also, for 9 instance, had representatives. We had a test working 10 11 group. There were AP&L and B&W people on that, as well as probably Bechtel. 12 VOICE: Well. I still don't quite understand. Is 13 Creswell it a picture of anyone on the staff contacting B&W regarding 14 issues that would develop, or was there a management control 15 16 over it? Cavanauch VOICE: Well, of course, it all funneled. On the 17 plant staff, it funneled to the plant superintendent. He 10 was then -- of course, he delegated various responsibilities 19 to members of his staff. And of course, he was the key 20 individual. 21 22 Creswell VOICE: At what point would you have become involved in these communications? 23 VOICE: I would become involved if there was. 24 Cavanaugh-

25 let's say, an item that could not be resolved, an item that

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NRCmte 1 I had to have interface through the Little Rock support 2 group to B&W. Creswell VOICE: Okay, If I understand what you're saying, 3 if there was an issue that arose during the testing program 4 5 at ANO-1 that the plant superintendent could not resolve in 6 discussions with site B&W personnel, that issue would be 7 followed -- would be forwarded to you for further 8 resolution? Cavanaugh 9 VOICE: That's correct. 10 VOICE: Okay. Who did you report to? Creswell Cavanaugh 11 VOICE: At that time I reported to the director of 12 power production. 13 VOICE: When I say report to at that time, that is Creswell from approximately August of 1974 to May of 1975. And your 14 15 -10 Cavanaugh VOICE: Director of power production. VOICE: Who was he? 17 Creswell 18 Cavanaugh VOICE: Mr. James H. Woodward. Creswell VOICE: Mr. Woodward. 19 (Pause.) 20 21 Creswell VOICE: Mr. Cavanaugh, now I'd like for you to, if you would, describe any pressurizer level problems that 22 you're aware of that occurred at Arkansas Nuclear One. 23 24 starting as far back as you knew it could be a problem. Cavanaugh VOICE: Well. I think the one that became 25

NRCmte

evident and caused us to have extensive discussions with B&W . 1 occurred in, I believe, in the fall of '74, subsequent to 2 some reactor trips, where we lost pressurizer level 3 indication for a short period of time. And of course, we, 4 as a follow-up to that, had extensive discussions with B&W 5 related to why, of course, we were losing level, what was 6 different about Arkansas Nuclear One than previous -- I say 7 "previous"; previous B&W plants that had gone into 8 commercial operation; and was in fact there an unreviewed 9 safety question involved here. 10

11 These I believe first occurred in the fall of 12 '74.

VOICE: How was the problem resolved? 13 Creswell VOICE: Well, the problem was finally resolved 14 Cavanaugh with some basic operating -- to the best of my recollection. 15 it was resolved primarily with. one. to establish that there 10 was not a safety, unreviewed safety question, and this was 17 demonstrated at a point in time in these discussions by 18 19 B&W. who had done some extensive analyses and comparisons to other plants and similar trips and plant characteristics 20 21 after the trip.

It was also resolved from the standpoint of maintaining pressurizer level indication within the indicating band by changes basically in our operation and in our control systems.

25

1 Q NRCmte VOICE: Okay. Was this issue resolved to your 2 satisfaction? Cavanaugh VOICE: Yes, it was resolved to our satisfaction 3 4 and it took a while to be resolved, but yes, it was resolved 5 to our satisfaction. Creswell 6 VOICE: This is Jim Creswell speaking again. 7 Mr. Cavanaugh, I'm going to show you a document here. This document is a memorandum to a Mr. Oles with the 8 9 Babcock & Wilcox Corporation, who was a senior project manager at that time. The subject is Arkansas Nuclear Unit 10 .11 I, pressurizer level set point. And this memo was 12 apparently from you to Mr. Oles. Would you take a look at 13 it to refresh your memory? 14 (Pause.) 15 VOICE: Yes, sir. Cavanaugh 16 Creswell VOICE: In this memorandum, you note that in another letter -- this was a letter of J. Kaylin to 17 J. Anderson, dated September 26th, 1974 -- that a request 18 19 had been made that the operating procedures at ANO-1 be 20 modified to increase the normal operating pressurizer level 21 from 180 inches to 210 inches. 22 You point out that if this change is made, that some operating experience at TMI-I has shown that, with the 23 24 level increases being experienced during a transient there,

that you might lose pressurizer level indication high. You

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NRCmte	i	further state that you cannot implement the change until
	2	further justification and resolution of the following
	3	comments is provided. Those comments are:
	4	That no analysis has been provided to AP&L to
	5	verify that this change will not cause the pressurizer to
	6	fill solid, resulting in loss of level indication
	7	(Inaudible) without reactor trip;
	8	Two, no analysis results have been provided which
	9	indicate the accident analysis contained in the FSAR would
	10	not be affected;
	11	Three, by basing the recommendation on a transient
	12	including one turbine bypass valve partially open, it is not
	13	valid since the turbine bypass system is not (Inaudible) and
-	14	more than one valve may stick open, resulting in a more
•	15	severe transient;
	16	And four, explain the reason for the difference in
	17	the location of the level tap at ANO versus TMI-1 and Oconee
	18	1, 2 and 3.
	19	Were you ever provided with an analysis by B&W to
	20	justify that level increase?
	21	Cavanaugh VOICE: I believe, to the best of my recollection.
	22	that after subsequent discussions with B&W, that they
	23	changed their recommendation on changing that particular
-	24	level change.
•	25	Creswell VOICE: To your knowledge, did B&W ever do an

8630 01 11 12 analysis before making the recommendation to justify the NRCmte 1 2 increased level? VOICE: I don't know that. 3 Cavanaugh 4 VOICE: Were you ever provided an analysis 0 regarding more than one turbine bypass valve malfunctioning? 5 Cavanaugh 6 VOICE: I can't recall at this time whether we did 7 or not. Q 8 VOICE: Were you ever provided an explanation for 9 the reason for the difference in the location of the level 10 tap at ANO versus TMI-1 and Oconee 1, 2 and 3? Cavanaugh VOICE: I know that we had discussions with B&W 11 12 about that. In my understanding, it had something to do, I 13 believe, with the manufacturing. But I don't recall the 14 exact B&W reason for the change. 15 Creswell VOICE: Mr. Cavanaugh -- Jim Creswell speaking 16 again -- I'm going to show you a copy of a figure out of the 17 FSAR for ANO-1. It's entitled "Pressurizer Outline," and the figure number is 4-6. Would you take a look at that, 18 19 please, sir. 20 (Pause.) 21 Creswell VOICE: In your opinion and your professional 22 judgment, is that tap shown properly located on that 23 drawing? Cavanaugh VOICE: I don't know that I could make that 24

25 determination right now. I would say that the level

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NRCmte	1	sensing nozzle says typical of three, and I guess I'd have
	2	to go back to the point in time I assume that this was in
	3	the original well, I'm not sure whether it was in the
	4	PSAR or FSAR. Did you say what the
	5	Creswell VOICE: I believe that's the FSAR.
	6	Cavanaugh VOICE: I guess I'd have to go back and compare it
	7	to the as-built drawings.
	8	Creswell VOICE: Would you say that that drawing shows the
	9	tap to be located at the tangent line between the
	10	cylindrical body and the hemispherical lower head of the
	11	pressurizer?
	12	Cavanaugh VOICE: Yes, it shows it to be in that region. Of
	13	course, it says "typical." I'm not sure that that's meant
_	14	to indicate an exact location, seeing as how there are no
	15	dimensions on this outline.
	16	Creswell VOICE: Okay.
	17	Next I'm going to show you a copy of Table 7-11,
	18	which is entitled "Information Readouts Available to the
	19	Operator for Monitoring Conditions in the Reactor, Reactor
	20	Coolant System, and in the Containment." This is on page
	21	7-56 of the FSAR and it's Amendment No. 36 dated April 6th,
	22	1973.
	23	Would you take a look at that, please?
	24	(Pause.)
•	25	Cavanaugh VOICE: I couldn't verify that this is from our

NRCmte	1	FSAR without looking at it.
	2	Creswell VOICE: Sure.
	3	(Pause.)
	4	Creswell VOICE: As an item in this table, a major
	5	parameter, pressurizer level is indicated, and the
	6	indication range or indicator range is specified as zero to
	7	400 inches. To your knowledge, is that information correct?
	8	Cavanaugh VOICE: I don't know that I could say right now.
	9	I would assume that it was deemed to be correct when we put
	10	it in the FSAR.
	11	Creswell VOICE: To your knowledge, has it ever been
	12	reviewed or changed?
	13	Cavanaugh VOICE: I can't say that. I don't know. I don't
_	14	remember, since it was some time ago. That was 1973.
•	15	Creswell VOICE: Okay. Let me ask you this. Could I ask
	16	you to review your FSAR, the copy that you have here, the
	17	control copy, and see if it has been changed?
	18	Cavanaugh VOICE: Yes, I could do that.
	19	N VOICE: Why don't we go off the record, then.
	20	Cavanaugh VOICE: Okay.
	21	(Discussion off the record.)
	22	Cavanaugh VOICE: These are the same table and figure that's
	23	in there, my FSAR.
	24	Creswell VOICE: Is your FSAR a control copy?
•	25	Cavanaugh VOICE: Yes.

NRCmte	1	Q VOICE: It is? So that from that I would assume,
	2	unless there is an error in your control FSAR document, that
	3	these would be the latest up to date information?
	4	Cavanaugh VOICE: To my knowledge.
	5	Creswell VOICE: Okay. Next I'm going to show you a
	6	document here which is a piece of correspondence from the
	7	Babcock & Wilcox Company directed to you, dated November
	8	18th, 1974, from Mr. Bowes, the senior project manager
	9	but the memo was sent out, was signed by H.A. Baker, the
	10	project manager. The subject of this memorandum is Arkansas
	11	Nuclear One pressurizer level set point, B&W reference
	12	NSS-8.
	13	I'd like for you to take a look at this
	14	memorandum.
	15	(Pause.)
	16	Creswell VOICE: In reading this memorandum, I note that
	17	the recommendation for increasing the pressurizer level from
	18	180 to 210 inches was discussed. Basically, if I understand
	19	the memo correctly and if I'm wrong, would you correct me
	20	that they were withdrawing the recommendation.
	21	Cavanaugh VOICE: Yes, that's correct.
	22	Creswell VOICE: They further suggested that your operators
	23	be instructed to secure letdown flow and increase makeup
	24	flow immediately following a reactor trip, to help maintain
	25	pressurizer level.

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What did you do with this information that B&W

2 sent to you?

3 Cavanaugh VOICE: That was then forwarded on to the plant 4 for their evaluation. I believe also that this situation 5 was presented to continue the review by the plant safety 6 committee and also the safety review, the corporate safety 7 review committee.

I can't recollect about the letdown flow, but I can -- I do remember the fact that in fact we were manually initiating high pressure injection in order to maintain the level.

12 VOICE: Approximately when did you start that?
13 Cavanaugh VOICE: I don't recall the exact time. It may
14 have been before that letter.

15 Q VOICE: Would you have initiated that change 16 without a B&W recommendation? Would you have initiated it 17 on your own? By "your own," I'm saying the management 18 structure of Arkansas Power & Light.

19 Cavanaugh VOICE: Which change?

20 Q VOICE: That is, to decrease or stop letdown flow 21 and increase charging flow, starting --

22 Cavanaugh VOICE: Well, that would work, that would be a 23 change to a procedure. It would be reviewed by the plant 24 safety committee, and then if they felt that it presented an 25 unreviewed safety question or felt that they wanted the

NRCmte 1

1 safety review committee to review it, then it would be sent
2 to the safety review committee for their review.

3 Q VOICE: But to your recollection, was the decision 4 that was made to do this based upon this B&W recommendation 5 at this point of time?

6 Cavanaugh VOICE: I can't say that.

Q VOICE: Was that ultimately the corrective action
8 taken regarding the pressurizer level problem, that is, an
9 operator action?

10 Cavanaugh VOICE: I don't believe that was the ultimate, the 11 ultimate correction. The ultimate correction, I believe, 12 involved some changes in the control systems so that we 13 would not lose level indication. We had a concern, as I 14 recall, about getting too many cycles on the high pressure 15 injection nozzles, and we wanted to get into a mode, I 16 believe, where we did not have to do that.

17 Q VOICE: And that was done primarily by changes to 18 control systems?

19 Cavanaugh VOICE: Right. Whin I say control systems, I mean 20 I believe that we made, for instance, a change in the 21 integrated control system. The basic goal was to reduce the 22 pressurizer level decrease and those things that would cause 23 it, for instance, excessive blowback on the steam dump and 24 bypass system and steam safety valves.

25 Creswell VOICE: Next I'm going to show you a document

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dated December oth, 1974, from William Cavanaugh to
 G.M. Bowles, senior project manager at B&W. The subject is
 "Arkansas Nuclear One, Unit I, pressurizer level set point."
 Would you take a look at that to refresh your memory?
 (Pause.)

6 Creswell VOICE: In reviewing the text of this memorandum, 7 you state that previous correspondence had not addressed the 8 fact that the ANO lower level tap is 40 inches above the tap 9 on Oconee and Three Mile Island. You further stated that 10 you had problems with this arrangement, and:

II Item one, the FSAR Figure 4.6 shows the lower taps located below the heater bundles, near the bottom of the pressurizer, which is in conflict with the actual location.

Now, I've previously shown you Figure 4.6. In reading your statement here, it would tend to indicate that you did have a problem with the way that tap was illustrated on the figure.

18 Cavanaugh VOICE: Well, I do not remember the specific 19 reference about that being above or below the heater 20 bundle. I do recall that, of course, that we did note that 21 our lower level sensing nozzle was in fact 40 inches above 22 the -- what was, I guess, on TMI-1 and on Oconee; and we 23 wanted further explanation of that.

24 Creswell VOICE: I'm going to jump ahead in time a little. 25 There was a telephone call apparently on the 12th of

19 8630 02 01 . December, 1974, between yourself and Mr. Baker, who I assume NROnte 1 was with Babcock & Wilcox. and Mr. Ruiter. R-u-i-t-e-r. Do 2 you recall what the substance of that telephone conversation 3 4 was? VOICE: Not right offhand, no, I don't. 5 Cavanaugh VOICE: Do you recall whether or not the location 0 Creswell 7 of the lower taps on the pressurizer were discussed during 8 that conversation? Cavanaugh VOICE: No. I don't. 4 10 Creswell VOICE: In the two memorandum that I presented to you here today, the one dated December 6th and the one dated 11 12 October 18th, you have asked questions or showed concern about the location of the lower level taps. I'd like to 13 14 again ask you: Were you ever told by B&W or anyone why 15 those taps were 40 inches higher? Cavanauch VOICE: I'm sure -- well, I'm sure we were. Our 10 17 main concern was not that the tap was different; our main concern was to ensure that we did not have an unsafe 18 19 condition. And the location of the tap was just part of the

> 20 overall analysis and investigation that we were conducting 21 at the time.

22 Creswell VOICE: Mr. Cavanaugh, I'm asking it from this 23 perspective. Here you have a component installed in your 24 plant where a design change has been made, a design change 25 that has resulted in a concern, a significant -- a 8630 02 02

NRCmte 1 significant concern, as indicated by these memoranda. Yet I 2 don't quite understand - well, I could understand why you 3 would want to resolve the concern.

But it would seem to me to be of interest as towhy the taps were changed.

Cavanaugh VOICE: I think it's more important as to the 6 question of whether or not the condition of the plant on a 7 subsequent trip -- it seems to me to be more important than 8 where a tap was located. The location of the tap was just 4 one item that was part of an overall big picture thing, the 10 big picture being what happens to pressurizer level, what 11 12 happens to the reactor coolant system from a trip at high 13 power levels.

15 porer reverse.

14 Creswell VOICE: Mr. Cavanaugh, do you have any 15 documentation regarding that telephone conversation that was

16 held on the 12th of December?

17 Cavanaugh VOICE: I don't know if we do or not.

18 Q VOICE: I'd like for you, if you would, to conduct

19 a good faith search ---

20 Cavanaugh VOICE: What was the date of that?

21 Q VOICE: December 12th, 1974.

22 A VOICE: 1974. A telecom between --

23 Q VOICE: The parties indicated here are Baker,

24 Cavanaugh, and Ruiter.

25 A VOICE: Baker, B&W. Okay.

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VOICE: We'll check the files and see.

Creswell 3 VOICE: Getting back to the December 6th, 1974, memoranda and your problems with the location of the 4 5 level taps, item 2 states that, following a reactor trip 6 from 75 percent full power level, indication was lost for 45 7 seconds. This indicates that following trips from 100 8 percent full power, the level indication could be lost in 9 excess of one minute, which does not rrespond with the "lost momentarily" statement in reference one -- reference 10 11 one being the letter. Baker to Cavanaugh, dated November 12 18th, 1974.

You further state in the ending of the memorandum: Please review the above to determine what actions can be taken to resolve these items and provide us with your recommendations and reasons for that level tap discrepancy by Januar' 6th, 1975.

18 It would appear from reading this that you set a 19 deadline for their response to this particular item, your 20 concern about the location of the taps.

21 Cavanaugh VOICE: Well, I don't think it's the location of 22 the taps, again. Again, it's the concern about pressurizer 23 level indication, pressure, the plant's response after a 24 trip: not the business of -- not the primary concern of 25 where the tap is. No, that was not the primary concern.

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NRCmte	I	Q VOICE: Well, I was let me read this sentence
	2	again: Please review the above to determine what actions
	3	can be taken to resolve these items and provide us with your
•	4	recommendations and reasons for the level-tap discrepancy by
	5	January 6th, 1975.
	6	You specifically address level-tap
	7	Cavanaugh VOICE: That's right, but that's
	8	Q VOICE: discrepancy.
	9	Cavanaugh VOICE: that's "and the reasons." So I think
	10	you're taking just that part out of context. It was the
	11	overall big picture that we wanted additional information
	12	on.
	13	Q VOICE: Now, obviously they didn't meet that
-	14	B&W didn't meet that deadline of January 6th.
•	15	Cavanaugh VOICE: I don't know that. I can't say that they
	16	didn't today. I'd have to go back and research all the
4.2	17	files to say that they didn't. I can't make that
	18	statement.
	19	Q VOICE: We'd like for you to conduct that search
	20	of your files and determine if that information was or was
	21	not provided to you.
	22	A VOICE: That memo was what?
	23	Q VOICE: This is December 6th, 1974.
_	24	A VOICE: December 6.
•	25	(Pause.)