

PEBBLE SPRINGS
NOV 21, 1977

BEFORE THE ATOMIC SAFETY LICENSING BOARD

In The Matter of
Portland General
Electric Company

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Docket Nos:
50-514
50-515

NRC Staff's Responses To Board Questions
During The Pebble Springs Pre-Hearing
Conference Held on April 12, 1978

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Date: May 25, 1978

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offsite power is available, and assuming a single active failure. The applicant was notified of these requirements and was requested to meet them, as a minimum, and provide a commitment to meet them prior to issuance of a construction permit.

Question No. 7

Has the staff reviewed the November 30, 1977 applicant's response to a series of questions raised by the ACRS? Are there any unresolved questions at this time? Can construction proceed pending a resolution? Are all of them included in the staff's list of unresolved items?

With respect to Question No. 9 of the ACRS, on the definition of a single failure, does the staff agree that the criterion stated by the applicant for single failure are adequate in designing redundant safety systems, and should the same criteria apply to all safety-grade systems regardless of the challenge risk?

Response

The staff reviewed the applicant's responses to questions from the Advisory Committee on Reactor Safeguards and we reported at the meeting of January 6, 1978, that it found nothing that would change our evaluation as reported in the Safety Evaluation Report and the four supplements. The majority of the questions requested additional clarifications of specific items of interest. Some posed assumptions and scenarios which exceeded those criteria that are necessary for the protection of the health and safety of the public. We agreed with the applicant's response, where it has indicated, that the system was designed in accordance with the requirements of the Nuclear Regulatory Commission. During these discussions, two items of concern were raised by the Committee. These were the electrical penetration requirements and containment purging at power. These are now resolved to our satisfaction and reported in Supplement No. 5 to the Safety Evaluation Report.

Finally, the favorable letter of January 12, 1978 from the Advisory Committee on Reactor Safeguards indicates that the Committee was satisfied with the applicant's responses, including the additional information that was provided by the staff and applicant at the Pebble Springs ACRS review meeting.

Question No. 8

Does Supplement No. 4 to the Safety Evaluation Report represent the final staff evaluation of the safe shutdown earthquake, and did Supplement No. 4 include the ACRS and all other evaluations?