DUKE POWER COMPANY

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28201

A. C. THIES SENIOR VICE PRESIDENT PRODUCTION AND TRANSMISSION P. O. Box 2178

May 6, 1975

Mr. Norman C. Moseley, Director U. S. Nuclear Regulatory Commission Suite 818 230 Peachtree Street, Northwest Atlanta, Georgia 30303

Re: IE:II:TNE 50-269/75-1 50-270/75-1 50-287/75-1

Dear Mr. Moseley:

The purpose of this letter is to appraise you of additional information regarding Item I.B.3, "Personnel Radiation Exposure Records," of IE Inspection Report 50-269, -270, and -287/75-1. As you are aware, this item concerned the reduction of the exposure records of two individuals and not similarly reducing the records of all other personnel when a possible error in TLD dosimetry results was discovered. In the case of the two individuals noted, the TLD badges were significantly greater than those indicated by their dosimeter readings. This is a relatively infrequent occurrence as dosimeters usually indicate greater exposure than the TLD's. Subsequently, a letter received from the TLD vendor described uncertainties in the TLD results and concluded that the fourth quarter TLD readings had been reported as 15 percent to 30 percent high. Therefore, one individual, previously thought to have received a dose of 3040 millirem, was assigned an exposure of 2110 millirem based upon his film badge and dosimeter readings. The second individual, previously thought to have received a cose of 3410 millirem, was assigned an exposure of 2965 millirem based upon his dosimeter readings and a conservative TLD correction factor of 15 percent.

In order to provide consistent radiation exposure records, a review of all records of the fourth quarter is being performed. In instances where the dosimeters indicate significantly lower exposures than the TLD results, and will support the minimum correction factor of 15

Mr. Norman C. Moseley Page 2 May 6, 1975

percent suggested by the TLD vendor, the exposure records will be corrected by reducing the indicated exposure 15 percent. It is expected that these records will be corrected by June 15, 1975.

Very truly yours,

A. C. Thies

ACT:vr

UNITED STATES

NUCLEAR REGULATORY COMMISSION

REGION II

230 PEACHTREE STREET, N. W. SUITE 818

ATLANTA, GEORGIA 30303

MAY 2 7 1975

In Reply Refer To:

IE:II:TNE

50-269/75-1, 50-270/75-1

50-287/75-1

for encirones

Duke Power Company
ATTN: Mr. A. C. Thies
Senior Vice President
Production and Transmission
Power Building
422 South Church Street
Charlotte, North Carolina 28242

Gentlemen:

Thank you for your letters dated March 26, 1975, April 11, 1975, April 30, 1975, and May 6, 1975, informing us of steps you have taken to correct the items of noncompliance concerning activities under NRC Operating License Nos. DPR-38, DPR-47 and DPR-55, which were brought to your attention in our letter which transmitted IE Inspection Report Nos. 50-269, 50-270 and 50-287/75-1. We will examine your corrective actions and plans during subsequent inspections.

We have reviewed your response to item I.A.3 pertaining to annual surveillance scheduling on Oconee Unit 2. This item was referred to the Office of Inspection and Enforcement, Headquarters, for further evaluation. Since this matter is still under review, we have no further questions at this time.

We appreciate your cooperation with us.

Very truly yours,

Norman C. Moseley

Director





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422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28201

A C THIES SENIOR VICE PRESIDENT PRODUCTION AND TRANSMISSION

P. J. Box 2178

April 30, 1975

Mr. Norman C. Moseley, Director U. S. Nuclear Regulatory Commission Suite 818 230 Peachtree Street, Northwest Atlanta, Georgia 30303

Re: IE:II:TNE 50-269/75-1

50-270/75-1 50-287/75-1

Dear Mr. Moseley:

In my letter of March 26, 1975 a status of the resolution of problems associated with process radiation monitors at the Oconee Nuclear Station was provided. The purpose of this letter is to provide an update of this status.

1. Liquid Waste Process Monitors RIA-33 and RIA-34

The problems with these monitors have been resolved. Continued testing will be performed to assure proper operation of these monitors.

2. Iodine Monitors RIA-44 and -48

The problems associated with these monitors have been resolved. Continued testing will be performed to assure proper operation of these monitors.

3. Waste Gas Monitors RIA-37 and -38

A modification will be made to these monitors which will permit an air purge to clear the monitor after a waste gas decay tank release. In addition, a gas sample line will be added in parallel with the monitor to allow a grab sample to be taken at the monitor. It is expected that the necessary materials for this modification will be ordered in the near future and that the modification will be completed within the next several months.

Mr. Norman C. Moseley Page 2 April 30, 1975

4. Particulate Monitors RIA-43 and -47

The Unit 1 particulate monitor has been modified and is being evaluated for its effectiveness to provide better correlation between the monitor and grab samples. Data which has been collected to date has not been conclusive. It is expected that sufficient data will be collected to permit this determination by June 1, 1975.

We are continuing to actively pursue resolution of the outstanding problems associated with the process monitors. We are also continuing to provide appropriate monitoring to assure effluent releases are maintained within station limits and guidelines while the remaining problems are being resolved. We will provide you another status report by June 1, 1975.

Very truly yours,

A. C. Thies

ACT:vr

