## PORTLAND GENERAL ELECTRIC COMPANY

121 S.W. SALMON STREET

PORTLAND, OREGON 97204

CHARLES GOODWIN, JR.

December 29, 1978

Trojan Nuclear Plant Docket 50-344 License NPF-1

Director of Nuclear Reactor Regulation
ATTN: Mr. A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sir:

In a letter dated April 12, 1978 we advised that Westinghouse had notified Portland General Electric Company (PCE) of a logic inconsistency in the Loss-of-Coolant Accident Emergency Core Cooling System (ECCS) evaluation for the Trojan Nuclear Plant. At that time, we committed to provide a schedule for submitting for NRC approval a corrected ECCS calculation as soon as Westinghouse and the NRC had agreed upon an acceptable model. On December 20, 1978 the NRC issued an exemption to PGE to permit interim operation of Trojan until a corrected ECCS evaluation has been submitted and approved.

It is our understanding that either of two approaches are acceptable to the NRC. Namely, the improved "February 1978" Westinghouse ECCS evaluation model can be used, or, alternatively, the previous "October 1975" model can be used if it is appropriately corrected for the zirconium-water reaction error identified above. Westinghouse has advised PGE that their sensitivity studies indicate the "February 1978" ECCS evaluation model applied to Trojan would accommodate a total peaking factor (FQ) of 2.32 with a peak cladding temperature (PCC) ites than the 10 CFR 50.46 limit of 2200°F. In contrast, preliminary results of the Trojan analyses using the "October 1975" model corrected for the zirconium-water reaction error indicate that a peaking factor of 2.25 results in a PCT of 2142°F; with FQ = 2.32 the PCT would exceed 2200°F. The difference in the two results is due to the fact that the "February 1978" model has been refined in several areas over the "October 1975" model.

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PGE has elected to submit an ECCS evaluation using the "October 1975" Westinghouse ECCS evaluation model corrected for the zirconium-water reaction error. We will submit this analysis for NRC review and a request for license amendment for the F<sub>Q</sub> Technical Specification before April 1, 1978. In the interim until the NRC has reviewed the revised Trojan "October 1975" model ECCS analyses and issued an amendment, PGE will administratively limit the allowable F<sub>Q</sub> to 2-25, as illustrated in the attached marked up Trojan Technical Specifications.

Sincerely,

C. Goodwin, Jr.

Assistant Vice President
Thermal Plant Operation and
Maintenance

CG/CJP/4crw2A18

c: Director State of Oregon Department of Energy