



Public Service Company of Colorado

16805 ROAD 19½
PLATTEVILLE, COLORADO 80651

July 15, 1980
Fort St. Vrain
Unit No. 1
P-80212

Mr. Karl V. Seyfrit, Director
Nuclear Regulatory Commission
Region IV
Office of Inspection and Enforcement
611 Ryan Plaza Drive
Suite 1000
Arlington, Texas 76012

Reference: Facility Operating License
No. DPR-34

Docket No. 50-267

Dear Mr. Seyfrit:

Enclosed please find a copy of Reportable Occurrence Report No. 50-267/80-32, Final, submitted per the requirements of Technical Specification AC 7.5.2(b)2.

Also, please find enclosed one copy of the Licensee Event Report for Reportable Occurrence Report No. 50-267/80-32.

Very truly yours,

Don Warambourg
Don Warambourg
Manager, Nuclear Production

DW/clg

Enclosure

cc: Director, MIPC

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REPORT DATE: July 15, 1980

REPORTABLE OCCURRENCE 80-32
ISSUE 0

OCCURRENCE DATE: June 17, 1980

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FORT ST. VRAIN NUCLEAR GENERATING STATION
PUBLIC SERVICE COMPANY OF COLORADO
16805 WELD COUNTY ROAD 19 1/2
PLATTEVILLE, COLORADO 80651

REPORT NO. 50-267/80-32/03-L-0

Final

IDENTIFICATION OF
OCCURRENCE:

On June 17 and 18, 1980, the plant was operated at power with emergency feedwater header supply to Loop 1 helium circulator water turbine drives isolated.

This constitutes operation in a degraded mode of LCO 4.2.2(a), and is reportable per Fort St. Vrain Technical Specification AC 7.5.2(b)2.

EVENT
DESCRIPTION:

June 15, 1980, with the plant operating at approximately 68% thermal power and 193 MWe, a Plant Trouble Report was initiated stating that PV-21243 was not controlling at the appropriate setpoint. This problem did not render the emergency feedwater header supply to Loop 1 circulators inoperable.

Refer to Figure 1. The Loop 1 pressure control system consists of two valves (① and ②). Main pressure control valve PV-21243 (① of Figure 1) is designed to control feedwater supply pressure under flow conditions. Secondary valve PV-21243-1 (② of Figure 1) is provided to bleed off leakage from the main valve to the turbine water drain tank under the no-flow conditions which normally exist.

Preparations were being made to isolate the Loop 1 emergency feedwater header on June 17, 1980, for repair of a line leak; and PV-21243 was to be repaired at the same time. Fort St. Vrain Technical Specification LCO 4.2.2(a) provides that the emergency feedwater header may be isolated for up to 24 hours with the reactor at power without the helium circulators being considered inoperable. Prior to isolating the system on June 17, a loop shutdown resulted in a reactor power reduction to less than 2%.

EVENT

DESCRIPTION: (Cont'd)

The Loop 1 emergency feedwater header supply was isolated via V-211615 (③ of Figure 1) at 0830 hours on June 17 for repair of the problems noted above. Because the reactor was not at power, no LCO 4.2.2 degraded mode operation occurred. However, by 2115 hours all preparations for return to power operation had been completed, and reactor power was increased to greater than 2%. Because PV-21243 repair was not complete, the Loop 1 emergency feedwater supply was still isolated. This constitutes operation in a degraded mode of LCO 4.2.2(a).

By 1500 hours on June 18, 1980, repair work had been completed. The system was returned to service within the 24 hours allowed by LCO 4.2.2(a), thus meeting the requirements for circulator operability.

Had it been necessary during this period of degraded mode operation, the Loop 1 circulators could have been operated on water turbine drive utilizing a water supply from the emergency condensate or firewater systems.

CAUSE

DESCRIPTION:

Operation in LCO 4.2.2 degraded mode was the result of a return to reactor power operation with emergency feedwater header supply to Loop 1 circulators isolated.

System isolation was necessary to replace a leaking elbow on the 1" line downstream of PV-21243-1 and to repair PV-21243 to alleviate valve leakage. At the time the reactor was returned to power operation, the repair of PV-21243 was not complete; thus the system was still isolated.

CORRECTIVE

ACTION:

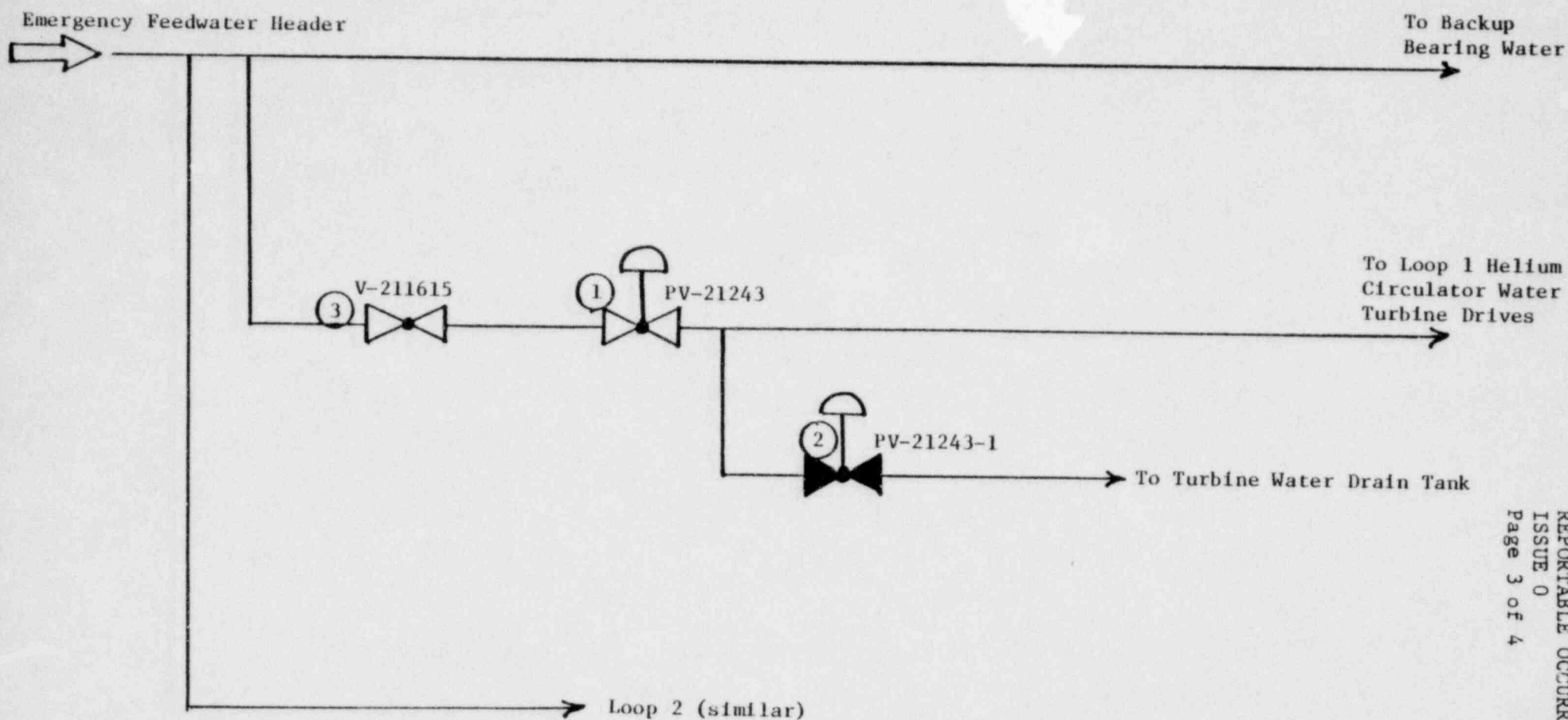
Problems with pressure control and leakage on PV-21243 were alleviated by applying devcon to the area under the seal ring gasket.

Valve repair was completed on June 18, 1980, and the system returned to service.

No further corrective action is anticipated or required.

FIGURE 1

Loop 1 Emergency Feedwater Pressure Control



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