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SHIELDS L. DALTROFF
VICE PRESIDENT
ELECTRIC PRODUCTION

July 2, 1980

Docket Nos.: 50-277
50-278

IE Bulletin 80-11

Mr. Boyce H. Grier, Director
Office of Inspection & Enforcement
Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Dear Mr. Grier:

This is in response to your letter of May 8, 1980, which forwarded IE Bulletin 80-11, and requires a response to items 1, 2a, and 3 of the Bulletin within 60 days of the issue date. A response to item 2b of the Bulletin will be forwarded within 180 days of the issue date of the Bulletin. The actions requested in items 1, 2a, and 3 of the Bulletin, and our responses are listed sequentially below.

Action to be Taken by Licensee:

1. Identify all masonry walls in your facility which are in proximity to or have attachments from safety-related piping or equipment such that wall failure could affect a safety-related system. Describe the systems and equipment, both safety and non-safety-related, associated with these masonry walls. Include in your review, masonry walls that are intended to resist impact or pressurization loads, such as missiles, pipe whip, pipe break, jet impingement, or tornado, and fire or water barriers, or shield walls. Equipment to be considered as attachments or in proximity to the walls shall include, but is not limited to, pumps, valves, motors, heat exchangers, cable trays, cable/conduit, HVAC ductwork, and electrical cabinets, instrumentation and controls. Plant surveys, if necessary, for areas inaccessible during normal

plant operation shall be performed at the earliest opportunity.

Response

All masonry walls at Peach Bottom Atomic Power Station which are in proximity to or have attachments from safety-related piping or equipment are identified in Appendix A. Proximity is defined as being within a distance equal to the wall height, such that wall failure could render the safety-related system inoperable. Safety-related systems and equipment associated with these masonry walls are also described in Appendix A, together with the wall location, its primary function (shield wall, partition, etc.), and the tentative priority for re-evaluation.

Non-safety-related systems or equipment such as conduits, cable trays etc. attached to these walls have been surveyed and all data related to these systems have been obtained in order to establish imposed loading on masonry walls for use during the re-evaluation analysis. The information shown in Appendix A was obtained by performing a plant survey.

Appendix B lists masonry walls which do not have safety-related systems/equipment either attached to them or in proximity. The information shown in Appendix B was obtained either by plant survey and/or by drawing review.

A visual, walk-through verification will be made to confirm the drawing review. Should this investigation identify any walls which affect safety-related systems or equipment, those walls will be surveyed in detail, added to Appendix A, and included in the re-evaluation program.

2. Provide a re-evaluation of the design adequacy of the walls identified in Item 1 above to determine whether the masonry walls will perform their intended function under all postulated loads and load combinations. In this regard, the NRC encourages the formation of an owners' group to establish both appropriate re-evaluation criteria and where necessary, a later confirmatory masonry test program to quantify the safety margins established by the re-evaluation criteria (this is discussed further in Item 3 below).

   a. Establish a prioritized program for the re-evaluation of the masonry walls. Provide a description of the program and a detailed schedule for completion of the re-evaluation for the categories in the program. The completion date of all re-evaluations should not be more than 180 days from the date of this Bulletin. A higher priority should be placed on the wall re-evaluations considering safety-related piping 2-1/2 inches or

greater in diameter, piping with support loads due to thermal expansion greater than 100 pounds, safety-related equipment weighing 100 pounds or greater, the safety significance of the potentially affected systems, the overall loads on the wall, and the opportunity for performing plant surveys and, if necessary, modifications in areas otherwise inaccessible. The factors described above are meant to provide guidance in determining what loads may significantly affect the masonry wall analyses.

Response

The re-evaluation program will consist of a structural analysis of the masonry walls identified in Appendix A using an appropriate re-evaluation criteria which will define loads considered, load combinations and allowable stresses.

The following parameters will be considered in establishing priorities for wall re-evaluation:

a) Walls which have larger number of safety-related systems on them or in their proximity.

b) Walls which are subjected to heavier loading, and

c) Walls which, in terms of height, length and thickness are judged to be more critical than others.

Tentative re-evaluation priorities are indicated for each wall in Appendix A. Priority 1 and Priority 2 indicate the walls which will be analyzed first, followed by Priority 3 and 4 in that order.

The re-evaluation of masonry walls designated as Priority 1 and Priority 2 is expected to be completed by mid or late September, 1980. Re-evaluation of Priority 3 and Priority 4 masonry walls is expected to be completed by mid or late October, 1980. A report of the re-evaluation will be submitted by November 4, 1980, as required by the Bulletin.

3. Existing test data or conservative assumptions may be used to justify the re-evaluation acceptance criteria if the criteria are shown to be conservative and applicable for the actual plant conditions. In the absence of appropriate acceptance criteria a confirmatory masonry wall test program is required by the NRC in order to quantify the safety margins inherent in the re-evaluation criteria. Describe in detail the actions planned and their schedule to justify the re-evaluation criteria used in Item 2. If a test program is necessary, provide your commitment for such a program and a

schedule for submittal of a description of the test program and a schedule for completion of the program. This test program should address all appropriate loads (seismic, tornado, missile, etc.). It is expected that the test program will extend beyond the 180 day period allowed for the other Bulletin actions. Submit the results of the test program upon its completion.

Response

Justification for re-evaluation criteria will be submitted with the re-evaluation report within 180 days of the date of the Bulletin. Justification will be based on reference to effective codes and established standards of practice related to concrete and masonry design typically used throughout the industry.

It is anticipated that such justification will be considered appropriate, except as required to determine project unique structural properties such as collar joint strength and any other properties for which test data is not available or cannot otherwise be determined. Should these unique properties need to be determined by testing, adequate test procedures will be developed as required.

Very truly yours,

Attachments:
Appendix A
Appendix B

cc: United States Nuclear Regulatory Commission
Office of Inspection and Enforcement
Division of Reactor Operations Inspection
Washington, DC 20555

COMMONWEALTH OF PENNSYLVANIA :

ss.

COUNTY OF PHILADELPHIA :

S. L. Daltroff, being first duly sworn, deposes and says:

That he is Vice President of Philadelphia Electric Company; that he has read the foregoing response to IE Bulletin 80-11 and knows the contents thereof; and that the statements and matters set forth therein are true and correct to the best of his knowledge, information and belief.

S. L. Daltroff

Subscribed and sworn to
before me this 3rd day
of July, 1980

Elizabeth H. Boyer
Notary Public

ELIZABETH H. BOYER
Notary Public, Phila., Phila. Co.
My Commission Expires Jan. 30, 1982

APPENDIX "A"

SUMMARY OF MASONRY WALLS (SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

(SEE PAGE 7 FOR NOTES)

SERIAL NO	WALL NO (I)	WALL DESCRIPTION			WALL'S PRIMARY FUNCTION	SAFETY RELATED SYSTEMS/EQUIPMENT ATTACHED TO WALL (II)	SAFETY RELATED SYSTEMS/EQUIPMENT IN PROXIMITY OF WALL (II)	TENTATIVE RE-EVALUATION PRIORITY
		BUILDING	ELEV.	LOCATION				
1	15.1	Radwaste	88-0 & 91-6	Standby Gas Treatment Room	Shielding	7	Standby Gas Treatment System (Fans, Filters and Ducts)	3
2	15.2	-DO-	-DO-	-DO-	-DO-	7	-DO-	3
3	16.1	-DO-	-DO-	Reactor Core Isolation Cooling Pump Area, Unit 2	-DO-	None	Reactor Core Isolation Cooling Pump Turbine, Unit 2	3
4	16.2	-DO-	-DO-	-DO-, Unit 3	-DO-	None	-DO-, Unit 3	3
5	16.3	-DO-	-DO-	Waste Sludge Tank Room	-DO-	None	Safeguard Related Conduit	3
6	32.1	-DO-	116-0	Condensate Tank & Pump Rooms	-DO-	7, 10	None	3
7	32.2	-DO-	-DO-	-DO-	-DO-	None	480V AC Power	3
8	32.3	-DO-	-DO-	Condensate Pump Area	-DO-	1, 7	SGT & RCIC	3
9	32.4	-DO-	-DO-	-DO-	-DO-	-DO-	-DO-	3
10	32.5	-DO-	-DO-	-DO-	-DO-	-DO-	-DO-	3
11	32.10	-DO-	-DO-	Filter Holding Pump Room & Fuel Pool Filter Demineralizer Area	Shielding	7	None	3
12	32.11	-DO-	-DO-	-DO-	-DO-	None	7	3
13	32.12	-DO-	-DO-	-DO-	-DO-	None	7	3
14	56.1	-DO-	135-0	RW Control Room & Filter Aid Tank & Pump Area	-DO-	6	None	3
15	78.3	-DO-	165-0	Radwaste Building - Fan Room	-DO-	17	Emerg. Swgr Filter Housing	3
16	25.1	Turbine (Control Structure)	116-0	Switchgear & Conventional Chem. Lab	Partition	2, 3, 8 thru 11	High Pressure and Emergency Service Water Lines and Safeguard Conduits	2
17	25.2	-DO-	-DO-	-DO-	-DO-	2, 3	-DO-	2
18	40.1	-DO-	135-0	Battery Room, Unit 2	Fire Resistance	8, 10, 12, 14	Battery Rack, Channels A & C, Unit 2	1

SUMMARY OF MASONRY WALLS (SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL NO	WALL NO (I)	WALL DESCRIPTION			WALL'S PRIMARY FUNCTION	SAFETY RELATED SYSTEMS/EQUIPMENT ATTACHED TO WALL (II)	SAFETY RELATED SYSTEMS/EQUIPMENT IN PROXIMITY OF WALL (II)	TENTATIVE RE-EVALUATION PRIORITY
		BUILDING	ELEV.	LOCATION				
19	40.2	Turbine (Control Structure)	135-0	Switchgear Room Unit 2	Fire Resistance	2, 7, 9, 10, 12, 13, 14, 17	4 kV Switchgear, Channel C, & MCC Channel C, Unit 2	1
20	40.3	-DO-	-DO-	-DO-	-DO-	1, 2, 3, 6 thru' 14, 17.	4 kV Switchgear, Channel D, Unit 2	1
21	40.4	-DO-	-DO-	-DO-, Unit 3	-DO-	2, 8, 9, 11, 12,13, 17	-DO-, Channel D, Unit 3	1
22	40.5	-DO-	-DO-	-DO-, Unit 3	-DO-	2, 3, 5, 6, 7, 9, 10, 12, 13, 14, 17, 18	-DO-, Channel C, Unit 3	1
23	40.6	-DO-	-DO-	Battery Room, Unit 3	-DO-	1, 8, 10, 12, 14	Battery Rack, Channels A & C, Unit 3	1
24	40.7	-DO-	-DO-	Battery Room, Unit 2	-DO-	10, 12, 14	Battery Rack, Channels A, B, C, & D, Unit 2 and Fuse Boxes Channels A&C.	1
25	40.8	-DO-	-DO-	Switchgear Room, Unit 2	-DO-	7, 10, 12, 14	4 kV Switchgear, Channels A & C, Unit 2	1
26	40.9	-DO-	-DO-	-DO-, Unit 2	-DO-	7, 9 thru' 13	-DO-, Channels B & D, Unit 2	1
27	40.10	-DO-	-DO-	-DO-, Unit 3	-DO-	8, 9, 10, 17	-DO-, Channels B & D, Unit 3	1
28	40.11	-DO-	-DO-	-DO-, Unit 3	-DO-	2, 3, 7 thru' 12, 14, 17	-DO-, Channels A & C, Unit 3	1
29	40.12	-DO-	-DO-	Battery Room, Unit 3	-DO-	1, 8, 10, 12	Battery Rack, Channels A, B, C, & D, Unit 3 and Fuse Boxes Channels A&C	1
30	40.13	-DO-	-DO-	Battery Room & Switchgear, Unit 2	-DO-	6, 8 thru' 12, 14, 17	Battery Rack, Channels A & C, MCC Channel C, Fuse Boxes Channels, A & C 4 kV Switchgear, Channel C, Unit 2	1
31	40.14	-DO-	-DO-	-DO-	-DO-	2, 8, 10, 11, 12, 14	Battery Rack, Channels B & D, MCC Channel A, Fuse Boxes Channels B & D 4 kV Switchgear, Channel A, Unit 2	1
32	40.15	-DO-	-DO-	Switchgear Room, Unit 2	-DO-	2, 9, 11, 12	4 kV Switchgear, Channels C & D, Unit 2	1
33	40.16	-DO-	-DO-	Switchgear Room, Unit 2	-DO-	2, 8 thru' 14	4 kV Switchgear, Channels A & B, Channel B, Unit 2	1
34	40.17	-DO-	-DO-	Switchgear Room, Units 2 & 3	-DO-	2, 8 thru' 13	4 kV Switchgear, Channels D & D, Units 2 & 3	1

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL NO	WALL NO (I)	WALL DESCRIPTION			WALL'S PRIMARY FUNCTION	SAFETY RELATED SYSTEMS/EQUIPMENT ATTACHED TO WALL (II)	SAFETY RELATED SYSTEMS/EQUIPMENT IN PROXIMITY OF WALL (II)	TENTATIVE RE-EVALUATION PRIORITY
		BUILDING	ELEV.	LOCATION				
35	40.18	Turbine (Control Structure)	135-0	Switchgear Room, Units 2 & 3	Fire Resistance	2, 9, 11, 12	4 kV Switchgear, Channels B & B, Units 2 & 3,	1
36	40.19	-DO-	-DO-	Switchgear Room, Unit 3	-DO-	8, 9, 11, 12, 17	-DO-, Channels C & D, MCC Channel D, Unit 3	1
37	40.20	-DO-	-DO-	-DO-	-DO-	2, 7, 8, 9, 11 thru' 14	-DO-, Channels A & B, MCC Channel B, Unit 3	1
38	40.21	-DO-	-DO-	Switchgear & Battery Rooms, Unit 3	-DO-	1, 7, 8, 12, 14	Battery Rack, Channels A & C, Fuse Boxes Channels A &C 4 kV Switchgear, Channel C, Unit 3	1
39	40.22	-DO-	-DO-	-DO-	-DO-	4, 8, 10, 11, 12, 14	Battery Rack, Channels B & D, MCC Channel A, Fuse Boxes, Channels B & D 4 kV Switchgear, Channel A, Unit 3	1
40	40.23	-DO-	-DO-	Switchgear Room, Unit 2	-DO-	None	4 kV Switchgear, Channel D, Unit 2 and H&V Duct	3
41	40.24	-DO-	-DO-	-DO-, Unit 2	-DO-	None	-DO-	3
42	40.25	-DO-	-DO-	-DO-, Unit 3	-DO-	None	-DO-, Unit 3	3
43	40.26	-DO-	-DO-	-DO-, Unit 3	-DO-	None	-DO-	3
44	40.27	-DO-	-DO-	-DO-, Unit 2	-DC	None	4 kV Switchgear & MCC Channel A, Unit 2	3
45	40.28	-DO-	-DO-	-DO-	-DO-	None	4 kV Switchgear & MCC Channel B, Unit 2	3
46	40.29	-DO-	-DO-	-DO-, Unit 3	-DO-	None	4 kV Switchgear & MCC Channel B, Unit 3	3
47	40.30	-DO-	-DO-	-DO-	-DO-	None	4 kV Switchgear & MCC Channel A, Unit 3	3
8	68.1	-DO-	150-0	Cable Spreading Room	-DO-	1, 8, 9, 11, 13, 17	Safeguard Trays	1
9	68.2	-DO-	-DO-	-DO-	-DO-	1	-DO-	2
	68.3	-DO-	-DO-	-DO-	-DO-	1 thru'5, 7 thru'12, 14, 18	-DO-	1
	68.4	-DO-	-DO-	-DO-	-DO-	1 thru'7, 9 thru'17	-DO-	1
	68.5	-DO-	-DO-	-DO-	Shielding	None	-DO-	2

APPENDIX "A"

SUMMARY OF MASONRY WALLS (SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL NO	WALL NO (I)	WALL DESCRIPTION			WALL'S PRIMARY FUNCTION	SAFETY RELATED SYSTEMS/EQUIPMENT ATTACHED TO WALL (II)	SAFETY RELATED SYSTEMS/EQUIPMENT IN PROXIMITY OF WALL (II)	TENTATIVE RE-EVALUATION PRIORITY
		BUILDING	ELEV.	LOCATION				
53	71.1	Turbine (Control Structure)	165-0	Control Room	Shielding	None	Control Panels, Safeguard Trays	2
54	45.1	Reactor, Unit 2	135-0	Isolation Valve Comp.	-DO-	2, 3, 5	a) Containment Isolation Valves and Assoc. Conduits b) CRD Insert and Withdrawal Lines	1
55	45.2	-DO-	-DO-	-DO-	-DO-	2, 3, 4, 10, 15	-DO-	1
56	45.4	-DO-	-DO-	Steam Pipe Tunnel	-DO-	5, 15, 18	MSIV on Steam Line "A" and Related Conduits	2
57	45.6	-DO-	-DO-	-DO-	-DO-	None	5, 15, 18	2
58	75.6	-DO-	165-0	Isolation Valve Area	-DO-	None	Containment Isolation Valves and conduits & RWCU leakage detection temp. element	4
59	76.6	-DO-	-DO-	Load Centers	Fire Resistance	9, 10	Electrical Load Center	4
60	76.7	-DO-	-DO-	-DO-	-DO-	None	-DO-, 1 thru 7, 9, 10, 15, 17	1
61	76.8	-DO-	-DO-	-DO-	-DO-	None	Electrical Load Center	4
62	76.9	-DO-	-DO-	-DO-	-DO-	None	-DO-, 1 thru 7, 9, 10, 15	1
63	76.10	-DO-	-DO-	-DO-	-DO-	2	Electrical Load Center	4
64	87.1	-DO-	180-0	Isolation Valve Compartment	Shielding	None	Containment Isolation Valves, RWCU Leakage Detection Temp. Element	4
65	102.8	-DO-	234-0	Refueling Floor	-DO-	6	None	4
66	102.9	-DO-	-DO-	-DO-	-DO-	6	None	2
67	406.1	Reactor, Unit 3	135-0	Isolation Valve Comp.	-DO-	2, 3, 15	a) Containment Isolation Valves & Conduits b) CRD insert and Withdrawal Lines	1
68	406.2	-DO-	-DO-	-DO-	-DO-	2 thru 5, 7, 12	-DO-	1

APPENDIX A

SUMMARY OF MASONRY WALLS (SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL NO	WALL NO (I)	WALL DESCRIPTION			WALL'S PRIMARY FUNCTION	SAFETY RELATED SYSTEMS/EQUIPMENT ATTACHED TO WALL (II)	SAFETY RELATED SYSTEMS/EQUIPMENT IN PROXIMITY OF WALL (II)	TENTATIVE RE-EVALUATION PRIORITY
		BUILDING	ELEV.	LOCATION				
69	406.6	Reactor, Unit 3	135-0	Steam Pipe Tunnel	Shielding	None	MSIV on Steam Line "A" and Related Conduits	4
70	406.9	-DO-	-DO-	-DO-	-DO-	5, 12	MSIV on Steam Line "A" and Related Conduits	2
71	406.10	-DO-	-DO-	-DO-	-DO-	3, 5, 10, 12	-DO-	1
72	409.7	-DO-	165-0	Isolation Valve Area	-DO-	None	Containment Isolation Valve & Associated Conduits and RWCU Leakage Detection Temperature Element	4
73	410.1	-DO-	-DO-	Regenerative Heat Exchanger Area	-DO-	5	None	4
74	410.6	-DO-	-DO-	Load Center	Fire Resistance	None	Electrical Load Centers, 7,10,11	2
75	410.7	-DO-	-DO-	-DO-	-DO-	None	-DO-, 2,3,5,6,7,10,11,14,15	1
76	410.8	-DO-	-DO-	-DO-	-DO-	None	-DO-, 10,12	2
77	410.9	-DO-	-DO-	-DO-	-DO-	None	-DO-, 2,3,5,7,10,11,12,14,17	1
78	410.10	-DO-	-DO-	-DO-	-DO-	None	-DO-, 10,11,17	2
79	412.1	-DO-	180-0	Isolation Valve Compartment	Shielding	None	Containment Isolation Valve and Conduits & RWCU Leakage Detection Temp. Element	4
80	418.10	-DO-	234-0	Refueling Floor	-DO-	6	None	4
81	418.11	-DO-	-DO-	-DO-	-DO-	6	-DO-	4
82	532.1	Emergency Cooling Tower	158-0	Emergency Cooling Tower	Fire Resistance	4,8	Emergency Cooling Water Pump and Load Centers and Panels, MCC	4

APPENDIX "A"

SUMMARY OF MASONRY WALLS (SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL NO	WALL NO (I)	WALL DESCRIPTION			WALL'S PRIMARY FUNCTION	SAFETY RELATED SYSTEMS/EQUIPMENT ATTACHED TO WALL (II)	SAFETY RELATED SYSTEMS/EQUIPMENT IN PROXIMITY OF WALL (II)	TENTATIVE RE-EVALUATION PRIORITY
		BUILDING	ELEV.	LOCATION				
83	532.2	Emergency Cooling Tower	158-0	Emergency Cooling Tower	Fire Resistance	8	Load Centers and Panels, MCC	4
84	532.3	-DO-	-DO-	-DO-	-DO-	8	-DO-	4
85	C86.1	Circulating Water Pump Structure	112-0	Pump Room	-DO-	None	Emergency Service Water Piping	4
86	C86.2	-DO-	-DO-	-DO-	-DO-	None	-DO-	4

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APPENDIX "A"

SUMMARY OF MASONRY WALLS (SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2&3

NOTES:

I. Each wall is designated by a unique number such that the first part indicates the Civil/Structural Drawing number and the second part gives the wall number on that particular drawing, e.g. wall No. 45.2 indicates wall number 2 on Civil/Structural Drawing No. 45.

II. Safety related systems are designated as below:

1. Reactor Core Isolation Cooling (RCIC)
2. Reactor Heat Removal (RHR)
3. Core Spray
4. High Pressure Coolant Injection (HPCI)
5. Primary Containment Isolation
6. Process/Area Radiation Monitoring
7. Standby Gas Treatment
8. Emergency Service Water (ESW)
9. Emergency Diesel Generator
10. Safeguard 480 VAC Power (MCC, LC)
11. Safeguard 4 kV Power
12. Safeguard 125/250 VDC Power
13. Safeguard 120 VAC Power
14. Safeguard 24 VDC Power
15. Containment Atmospheric Dilution (CAD)
16. Reactor Protection System (RPS)
17. Safeguard HVAC System
18. Steam Leak Detection System

APPENDIX "B"

SUMMARY OF MASONRY WALL (NON-SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL NO.	WALL NO.	WALL DESCRIPTION			BASIS FOR CLASSIFICATION
		BUILDING	ELEVATION	LOCATION	
1	24.1	Turbine	116-0	Valve Operating Area	Plant Walkdown & Eng. Review
2	32.6	Radwaste	-DO-	Condensate Pump & Tank Rooms	Engineering Review
3	32.7	-DO-	-DO-	-DO-	-DO-
4	32.8	-DO-	-DO-	-DO-	-DO-
5	32.9	-DO-	-DO-	-DO-	-DO-
6	61.1	-DO-	150-0	Hopper Compartment	-DO-
7	61.2	-DO-	-DO-	-DO-	-DO-
8	61.3	-DO-	-DO-	Radwaste DH&V Equipment Compartment	-DO-
9	61.4	-DO-	-DO-	-DO-	-DO-
10	61.5	-DO-	-DO-	-DO-	-DO-
11	61.6	-DO-	-DO-	-DO-	-DO-
12	61.7	-DO-	-DO-	-DO-	-DO-
13	61.8	-DO-	-DO-	-DO-	-DO-
14	78.1	-DO-	165-0	Centrifuge Room	-DO-

APPENDIX "B"

SUMMARY OF MASONRY WALL (NON-SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL NO.	WALL NO.	WALL DESCRIPTION			BASIS FOR CLASSIFICATION
		BUILDING	ELEVATION	LOCATION	
15	78.2	Radwaste	165-0	Centrifuge Room	Engineering Review
16	45.3	Reactor, Unit 2	135-0	Steam Pipe Tunnel	Plant Walkdown & Eng. Rev.
17	45.5	-DO-	-DO-	-DO-	-DO-
18	75.1	-DO-	165-0	Clean Up Recirc. Pump Room	-DO-
19	75.2	-DO-	-DO-	-DO-	-DO-
20	75.3	-DO-	-DO-	-DO-	-DO-
21	75.4	-DO-	-DO-	-DO-	-DO-
22	75.5	-DO-	-DO-	-DO-	-DO-
23	75.7	-DO-	-DO-	-DO-	-DO-
24	76.1	-DO-	-DO-	Regen. Heat Exchanger	-DO-
25	76.2	-DO-	-DO-	Non Regen. Heat Exch. Room	-DO-
26	76.3	-DO-	-DO-	-DO-	-DO-
27	76.4	-DO-	-DO-	Transfer Pump Room	-DO-
28	76.5	-DO-	-DO-	Backwash Rec. Tank Room	Plant Walkdown & Eng. Review

APPENDIX "B"

SUMMARY OF MASONRY WALL (NON-SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL NO.	WALL NO.	WALL DESCRIPTION			BASIS FOR CLASSIFICATION
		BUILDING	ELEVATION	LOCATION	
29	76.11	Reactor, Unit 2	165-0	Transfer Pump Room	Engineering Review
30	97.1	-DO-	195-0	Prefilter & Filter Compartments	Plant Walkdown & Eng. Review
31	97.2	-DO-	-DO-	-DO-	Plant Walkdown
32	97.3	-DO-	-DO-	-DO-	-DO-
33	97.4	-DO-	-DO-	-DO-	-DO
34	101.1	-DO-	234-0	Refueling Floor	-DO-
35	102.1	-DO-	-DO-	-DO-	-DO-
36	102.2	-DO-	-DO-	-DO-	-DO-
37	102.3	-DO-	-DO-	-DO-	-DO-
38	102.4	-DO	-DO-	-DO-	-DO-
39	102.5	-DO-	-DO-	-DO-	-DO-
40	102.6	-DO-	-DO-	-DO-	-DO-
41	102.7	-DO-	-DO-	-DO-	-DO-
42	102.10	-DO-	-DO-	-DO	-DO-

APPENDIX "B"

SUMMARY OF MASONRY WALL (NON-SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL NO.	WALL NO.	WALL DESCRIPTION			BASIS FOR CLASSIFICATION
		BUILDING	ELEVATION	LOCATION	
43	102.11	Reactor, Unit 2	234-0	Refueling Floor	Plant Walkdown
44	406.3	Reactor, Unit 3	135-0	Neutron Monitoring Room	Plant Walkdown & Eng. Rev.
45	406.4	-DO-	-DO-	-DO-	-DO-
46	406.5	-DO-	-DO-	Steam Pipe Tunnel	-DO-
47	406.7	-DO-	-DO-	-DO-	-DO-
48	406.8	-DO-	-DO-	-DO-	-DO-
49	409.1	-DO-	165-0	Clean Up Recirc. Pump Room	-DO-
50	409.2	-DO-	-DO-	-DO-	-DO-
51	409.3	-DO-	-DO-	-DO-	-DO-
52	409.4	-DO-	-DO-	-DO-	-DO-
53	409.5	-DO-	-DO-	-DO-	-DO-
54	409.6	-DO-	-DO-	-DO-	-DO-
55	410.2	-DO-	-DO-	Backwash Rec.Tank Room	Engineering Review
56	410.3	-DO-	-DO-	Transfer Pump Room	-DO-

APPENDIX "B"

SUMMARY OF MASONRY WALL (NON-SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL NO.	WALL NO.	WALL DESCRIPTION			BASIS FOR CLASSIFICATION
		BUILDING	ELEVATION	LOCATION	
57	410.4	Reactor, Unit 3	165-0	Non Regen. Heat Exch. Room	Plant Walkdown & Eng. Rev.
58	410.5	-DO-	-DO-	-DO-	-DO-
59	410.11	-DO-	-DO-	Transfer Pump Room	-DO-
60	413.1	-DO-	195-0	Storage Area	Plant Walkdown
61	413.2	-DO-	-DO-	-DO-	-DO-
62	413.3	-DO-	-DO-	-DO-	-DO-
63	413.4	-DO-	-DO-	-DO-	-DO-
64	413.5	-DO-	-DO-	-DO-	-DO-
65	413.6	-DO-	-DO-	-DO-	-DO-
66	414.1	-DO-	-DO-	Prefilter & Filter Compartment	Plant Walkdown & Engr. Review
67	414.2	-DO-	-DO-	-DO-	Plant Walkdown
68	414.3	-DO-	-DO-	-DO-	-DO-
69	414.4	-DO-	-DO-	-DO-	-DO-
70	417.1	-DO-	234-0	Refueling Area	-DO-

APPENDIX "B"

SUMMARY OF MASONRY WALL (NON-SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL NO.	WALL NO.	WALL DESCRIPTION			BASIS FOR CLASSIFICATION
		BUILDING	ELEVATION	LOCATION	
71	418.1	Reactor, Unit 3	234-0	Refueling Area	Plant Walkdown
72	418.2	-DO-	-DO-	-DO-	-DO-
73	418.3	-DO-	-DO-	-DO-	-DO-
74	418.4	-DO-	-DO-	-DO-	-DO-
75	418.5	-DO-	-DO-	-DO-	-DO-
76	418.6	-DO-	-DO-	-DO-	-DO-
77	418.7	-DO-	-DO-	-DO-	-DO-
78	418.8	-DO-	-DO-	-DO-	-DO-
79	418.9	-DO-	-DO-	-DO-	-DO-
80	418.12	-DO-	-DO-	-DO-	-DO-
81	418.13	-DO-	-DO-	-DO-	-DO-
*					

* NOTE: Based on Engineering review, all masonry walls of Recombiner Building and masonry walls of Radwaste, Turbine, and Reactor Building (Units 2 & 3) not listed above, are non-safety related.

CC7521

CENTRAL FILES

PHILADELPHIA ELECTRIC COMPANY

2301 MARKET STREET

P.O. BOX 8699

PHILADELPHIA, PA. 19101

(215) 841-5001

SHIELDS L. DALTROFF
VICE PRESIDENT
ELECTRIC PRODUCTION

July 2, 1980

Docket Nos.: 50-277
50-278

IE Bulletin 80-11

Mr. Boyce H. Grier, Director
Office of Inspection & Enforcement
Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Dear Mr. Grier:

This is in response to your letter of May 8, 1980, which forwarded IE Bulletin 80-11, and requires a response to items 1, 2a, and 3 of the Bulletin within 60 days of the issue date. A response to item 2b of the Bulletin will be forwarded within 180 days of the issue date of the Bulletin. The actions requested in items 1, 2a, and 3 of the Bulletin, and our responses are listed sequentially below.

Action to be Taken by Licensee:

1. Identify all masonry walls in your facility which are in proximity to or have attachments from safety-related piping or equipment such that wall failure could affect a safety-related system. Describe the systems and equipment, both safety and non-safety-related, associated with these masonry walls. Include in your review, masonry walls that are intended to resist impact or pressurization loads, such as missiles, pipe whip, pipe break, jet impingement, or tornado, and fire or water barriers, or shield walls. Equipment to be considered as attachments or in proximity to the walls shall include, but is not limited to, pumps, valves, motors, heat exchangers, cable trays, cable/conduit, HVAC ductwork, and electrical cabinets, instrumentation and controls. Plant surveys, if necessary, for areas inaccessible during normal

plant operation shall be performed at the earliest opportunity.

Response

All masonry walls at Peach Bottom Atomic Power Station which are in proximity to or have attachments from safety-related piping or equipment are identified in Appendix A. Proximity is defined as being within a distance equal to the wall height, such that wall failure could render the safety-related system inoperable. Safety-related systems and equipment associated with these masonry walls are also described in Appendix A, together with the wall location, its primary function (shield wall, partition, etc.), and the tentative priority for re-evaluation.

Non-safety-related systems or equipment such as conduits, cable trays etc. attached to these walls have been surveyed and all data related to these systems have been obtained in order to establish imposed loading on masonry walls for use during the re-evaluation analysis. The information shown in Appendix A was obtained by performing a plant survey.

Appendix B lists masonry walls which do not have safety-related systems/equipment either attached to them or in proximity. The information shown in Appendix B was obtained either by plant survey and/or by drawing review.

A visual, walk-through verification will be made to confirm the drawing review. Should this investigation identify any walls which affect safety-related systems or equipment, those walls will be surveyed in detail, added to Appendix A, and included in the re-evaluation program.

2. Provide a re-evaluation of the design adequacy of the walls identified in Item 1 above to determine whether the masonry walls will perform their intended function under all postulated loads and load combinations. In this regard, the NRC encourages the formation of an owners' group to establish both appropriate re-evaluation criteria and where necessary, a later confirmatory masonry test program to quantify the safety margins established by the re-evaluation criteria (this is discussed further in Item 3 below).

   a. Establish a prioritized program for the re-evaluation of the masonry walls. Provide a description of the program and a detailed schedule for completion of the re-evaluation for the categories in the program. The completion date of all re-evaluations should not be more than 180 days from the date of this Bulletin. A higher priority should be placed on the wall re-evaluations considering safety-related piping 2-1/2 inches or

greater in diameter, piping with support loads due to thermal expansion greater than 100 pounds, safety-related equipment weighing 100 pounds or greater, the safety significance of the potentially affected systems, the overall loads on the wall, and the opportunity for performing plant surveys and, if necessary, modifications in areas otherwise inaccessible. The factors described above are meant to provide guidance in determining what loads may significantly affect the masonry wall analyses.

Response

The re-evaluation program will consist of a structural analysis of the masonry walls identified in Appendix A using an appropriate re-evaluation criteria which will define loads considered, load combinations and allowable stresses.

The following parameters will be considered in establishing priorities for wall re-evaluation:

a) Walls which have larger number of safety-related systems on them or in their proximity.

b) Walls which are subjected to heavier loading, and

c) Walls which, in terms of height, length and thickness are judged to be more critical than others.

Tentative re-evaluation priorities are indicated for each wall in Appendix A. Priority 1 and Priority 2 indicate the walls which will be analyzed first, followed by Priority 3 and 4 in that order.

The re-evaluation of masonry walls designated as Priority 1 and Priority 2 is expected to be completed by mid or late September, 1980. Re-evaluation of Priority 3 and Priority 4 masonry walls is expected to be completed by mid or late October, 1980. A report of the re-evaluation will be submitted by November 4, 1980, as required by the Bulletin.

3. Existing test data or conservative assumptions may be used to justify the re-evaluation acceptance criteria if the criteria are shown to be conservative and applicable for the actual plant conditions. In the absence of appropriate acceptance criteria a confirmatory masonry wall test program is required by the NRC in order to quantify the safety margins inherent in the re-evaluation criteria. Describe in detail the actions planned and their schedule to justify the re-evaluation criteria used in Item 2. If a test program is necessary, provide your commitment for such a program and a

schedule for submittal of a description of the test program and a schedule for completion of the program. This test program should address all appropriate loads (seismic, tornado, missile, etc.). It is expected that the test program will extend beyond the 180 day period allowed for the other Bulletin actions. Submit the results of the test program upon its completion.

Response

Justification for re-evaluation criteria will be submitted with the re-evaluation report within 180 days of the date of the Bulletin. Justification will be based on reference to effective codes and established standards of practice related to concrete and masonry design typically used throughout the industry.

It is anticipated that such justification will be considered appropriate, except as required to determine project unique structural properties such as collar joint strength and any other properties for which test data is not available or cannot otherwise be determined. Should these unique properties need to be determined by testing, adequate test procedures will be developed as required.

Very truly yours,

Attachments:
Appendix A
Appendix B

cc: United States Nuclear Regulatory Commission
Office of Inspection and Enforcement
Division of Reactor Operations Inspection
Washington, DC 20555

COMMONWEALTH OF PENNSYLVANIA :

ss.

COUNTY OF PHILADELPHIA :

S. L. Daltroff, being first duly sworn, deposes and says:

That he is Vice President of Philadelphia Electric Company; that he has read the foregoing response to IE Bulletin 80-11 and knows the contents thereof; and that the statements and matters set forth therein are true and correct to the best of his knowledge, information and belief.

S. L. Daltroff

Subscribed and sworn to
before me this 3rd day
of July, 1980

Elizabeth H. Boyer
Notary Public

ELIZABETH H. BOYER
Notary Public, Phila., Phila. Co.
My Commission Expires Jan. 30, 1982

SUMMARY OF MASONRY WALLS (SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

(SEE PAGE 7 FOR NOTES)

SERIAL NO	WALL NO (I)	WALL DESCRIPTION			WALL'S PRIMARY FUNCTION	SAFETY RELATED SYSTEMS/EQUIPMENT ATTACHED TO WALL (II)	SAFETY RELATED SYSTEMS/EQUIPMENT IN PROXIMITY OF WALL (II)	TENTATIVE RE-EVALUATION PRIORITY
		BUILDING	ELEV.	LOCATION				
1	15.1	Radwaste	88-0 & 91-6	Standby Gas Treatment Room	Shielding	7	Standby Gas Treatment System (Fans, Filters and Ducts)	3
2	15.2	-DO-	-DO-	-DO-	-DO-	7	-DO-	3
3	16.1	-DO-	-DO-	Reactor Core Isolation Cooling Pump Area, Unit 2	-DO-	None	Reactor Core Isolation Cooling Pump Turbine, Unit 2	3
4	16.2	-DO-	-DO-	-DO-, Unit 3	-DO-	None	-DO-, Unit 3	3
5	16.3	-DO-	-DO-	Waste Sludge Tank Room	-DO-	None	Safeguard Related Conduit	3
6	32.1	-DO-	116-0	Condensate Tank & Pump Rooms	-DO-	7, 10	None	3
7	32.2	-DO-	-DO-	-DO-	-DO-	None	480V AC Power	3
8	32.3	-DO-	-DO-	Condensate Pump Area	-DO-	1, 7	SGT & RCIC	3
9	32.4	-DO-	-DO-	-DO-	-DO-	-DO-	-DO-	3
10	32.5	-DO-	-DO-	-DO-	-DO-	-DO-	-DO-	3
11	32.10	-DO-	-DO-	Filter Holding Pump Room & Fuel Pool Filter Demineralizer Area	Shielding	7	None	3
12	32.11	-DO-	-DO-	-DO-	-DO-	None	7	3
13	32.12	-DO-	-DO-	-DO-	-DO-	None	7	3
14	56.1	-DO-	135-0	RW Control Room & Filter Aid Tank & Pump Area	-DO-	6	None	3
15	78.3	-DO-	165-0	Radwaste Building Fan Room	-DO-	17	Emerg. Swgr Filter Housing	3
16	25.1	Turbine (Control Structure)	116-0	Switchgear & Conventional Chem. Lab	Partition	2, 3, 8 thru 11	High Pressure and Emergency Service Water Lines and Safeguard Conduits	2
17	25.2	-DO-	-DO-	-DO-	-DO-	2, 3	-DO-	2
18	40.1	-DO-	135-0	Battery Room, Unit 2	Fire Resistance	8, 10, 12, 14	Battery Rack, Channels A & C, Unit 2	1

SUMMARY OF MASONRY WALLS (SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL NO	WALL NO (I)	WALL DESCRIPTION			WALL'S PRIMARY FUNCTION	SAFETY RELATED SYSTEMS/EQUIPMENT ATTACHED TO WALL (II)	SAFETY RELATED SYSTEMS/EQUIPMENT IN PROXIMITY OF WALL (II)	TENTATIVE RE-EVALUATION PRIORITY
		BUILDING	ELEV.	LOCATION				
19	40.2	Turbine (Control Structure)	135-0	Switchgear Room Unit 2	Fire Resistance	2, 7, 9, 10, 12, 13, 14, 17	4 kV Switchgear, Channel C, & MCC Channel C, Unit 2	1
20	40.3	-DO-	-DO-	-DO-	-DO-	1, 2, 3, 6 thru' 14, 17.	4 kV Switchgear, Channel D, Unit 2	1
21	40.4	-DO-	-DO-	-DO-, Unit 3	-DO-	2, 8, 9, 11, 12,13, 17	-DO-, Channel D, Unit 3	1
22	40.5	-DO-	-DO-	-DO-, Unit 3	-DO-	2, 3, 5, 6, 7, 9, 10, 12, 13, 14, 17, 18	-DO-, Channel C, Unit 3	1
23	40.6	-DO-	-DO-	Battery Room, Unit 3	-DO-	1, 8, 10, 12, 14	Battery Rack, Channels A & C, Unit 3	1
24	40.7	-DO-	-DO-	Battery Room, Unit 2	-DO-	10, 12, 14	Battery Rack, Channels A, B, C, & D, Unit 2 and Fuse Boxes Channels A&C.	1
25	40.8	-DO-	-DO-	Switchgear Room, Unit 2	-DO-	7, 10, 12, 14	4 kV Switchgear, Channels A & C, Unit 2	1
26	40.9	-DO-	-DO-	-DO-, Unit 2	-DO-	7, 9 thru' 13	-DO-, Channels B & D, Unit 2	1
27	40.10	-DO-	-DO-	-DO-, Unit 3	-DO-	8, 9, 10, 17	-DO-, Channels B & D, Unit 3	1
28	40.11	-DO-	-DO-	-DO-, Unit 3	-DO-	2, 3, 7 thru' 12, 14, 17	-DO-, Channels A & C, Unit 3	1
29	40.12	-DO-	-DO-	Battery Room, Unit 3	-DO-	1, 8, 10, 12	Battery Rack, Channels A, B, C, & D, Unit 3 and Fuse Boxes Channels A&C	1
30	40.13	-DO-	-DO-	Battery Room & Switch-gear, Unit 2	-DO-	6, 8 thru' 12, 14, 17	Battery Rack, Channels A & C, MCC Channel C, Fuse Boxes Channels, A & C 4 kV Switchgear, Channel C, Unit 2	1
31	40.14	-DO-	-DO-	-DO-	-DO-	2, 8, 10, 11, 12, 14	Battery Rack, Channels B & D, MCC Channel A, Fuse Boxes Channels B & D 4 kV Switchgear, Channel A, Unit 2	1
32	40.15	-DO-	-DO-	Switchgear Room, Unit 2	-DO-	2, 9, 11, 12	4 kV Switchgear, Channels C & D, Unit 2	1
33	40.16	-DO-	-DO-	Switchgear Room, Unit 2	-DO-	2, 8 thru' 14	4 kV Switchgear, Channels A & B, Channel B, Unit 2	1
34	40.17	-DO-	-DO-	Switchgear Room, Units 2 & 3	-DO-	2, 8 thru' 13	4 kV Switchgear, Channels D & D, Units 2 & 3	1

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL NO	WALL NO (I)	WALL DESCRIPTION			WALL'S PRIMARY FUNCTION	SAFETY RELATED SYSTEMS/EQUIPMENT ATTACHED TO WALL (II)	SAFETY RELATED SYSTEMS/EQUIPMENT IN PROXIMITY OF WALL (II)	TENTATIVE RE-EVALUATION PRIORITY
		BUILDING	ELEV.	LOCATION				
35	40.18	Turbine (Control Structure)	135-0	Switchgear Room, Units 2 & 3	Fire Resistance	2, 9, 11, 12	4 kV Switchgear, Channels B & B, Units 2 & 3,	1
36	40.19	-DO-	-DO-	Switchgear Room, Unit 3	-DO-	8, 9, 11, 12, 17	-DO-, Channels C & D, MCC Channel D, Unit 3	1
37	40.20	-DO-	-DO-	-DO-	-DO-	2, 7, 8, 9, 11 thru' 14	-DO-, Channels A & B, MCC Channel B, Unit 3	1
38	40.21	-DO-	-DO-	Switchgear & Battery Rooms, Unit 3	-DO-	1, 7, 8, 12, 14	Battery Rack, Channels A & C, Fuse Boxes Channels A & C 4 kV Switchgear, Channel C, Unit 3	1
39	40.22	-DO-	-DO-	-DO-	-DO-	4, 8, 10, 11, 12, 14	Battery Rack, Channels B & D, MCC Channel A, Fuse Boxes, Channels B & D 4 kV Switchgear, Channel A, Unit 3	1
40	40.23	-DO-	-DO-	Switchgear Room, Unit 2	-DO-	None	4 kV Switchgear, Channel D, Unit 2 and H&V Duct	3
41	40.24	-DO-	-DO-	-DO-, Unit 2	-DO-	None	-DO-	3
42	40.25	-DO-	-DO-	-DO-, Unit 3	-DO-	None	-DO-, Unit 3	3
43	40.26	-DO-	-DO-	-DO-, Unit 3	-DO-	None	-DO-	3
44	40.27	-DO-	-DO-	-DO-, Unit 2	-DO-	None	4 kV Switchgear & MCC Channel A, Unit 2	3
45	40.28	-DO-	-DO-	-DO-	-DO-	None	4 kV Switchgear & MCC Channel B, Unit 2	3
46	40.29	-DO-	-DO-	-DO-, Unit 3	-DO-	None	4 kV Switchgear & MCC Channel B, Unit 3	3
47	40.30	-DO-	-DO-	-DO-	-DO-	None	4 kV Switchgear & MCC Channel A, Unit 3	3
8	68.1	-DO-	150-0	Cable Spreading Room	-DO-	1, 8, 9, 11, 13, 17	Safeguard Trays	1
9	68.2	-DO-	-DO-	-DO-	-DO-	1	-DO-	2
0	68.3	-DO-	-DO-	-DO-	-DO-	1 thru'5, 7 thry'12, 14, 18	-DO-	1
	68.4	-DO-	-DO-	-DO-	-DO-	1 thru'7, 9 thru'17	-DO-	1
	68.5	-DO-	-DO-	-DO-	Shielding	None	-DO-	2

SUMMARY OF MASONRY WALLS (SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL NO	WALL NO (I)	WALL DESCRIPTION			WALL'S PRIMARY FUNCTION	SAFETY RELATED SYSTEMS/EQUIPMENT ATTACHED TO WALL (II)	SAFETY RELATED SYSTEMS/EQUIPMENT IN PROXIMITY OF WALL (II)	TENTATIVE RE-EVALUATION PRIORITY
		BUILDING	ELEV.	LOCATION				
53	71.1	Turbine (Control Structure)	165-0	Control Room	Shielding	None	Control Panels, Safeguard Trays	2
54	45.1	Reactor, Unit 2	135-0	Isolation Valve Comp.	-DO-	2, 3, 5	a) Containment Isolation Valves and Assoc. Conduits b) CRD Insert and Withdrawal Lines	1
55	45.2	-DO-	-DO-	-DO-	-DO-	2, 3, 4, 10, 15	-DO-	1
56	45.4	-DO-	-DO-	Steam Pipe Tunnel	-DO-	5, 15, 18	MSIV on Steam Line "A" and Related Conduits	2
57	45.6	-DO-	-DO-	-DO-	-DO-	None	5, 15, 18	2
58	75.6	-DO-	165-0	Isolation Valve Area	-DO-	None	Containment Isolation Valves and conduits & RWCU leakage detection temp. element	4
59	76.6	-DO-	-DO-	Load Centers	Fire Resistance	9, 10	Electrical Load Center	4
60	76.7	-DO-	-DO-	-DO-	-DO-	None	-DO-, 1 thru 7, 9, 10, 15, 17	1
61	76.8	-DO-	-DO-	-DO-	-DO-	None	Electrical Load Center	4
62	76.9	-DO-	-DO-	-DO-	-DO-	None	-DO-, 1 thru 7, 9, 10, 15	1
63	76.10	-DO-	-DO-	-DO-	-DO-	2	Electrical Load Center	4
64	87.1	-DO-	180-0	Isolation Valve Compartment	Shielding	None	Containment Isolation Valves, RWCU Leakage Detection Temp. Element	4
65	102.8	-DO-	234-0	Refueling Floor	-DO-	6	None	4
66	102.9	-DO-	-DO-	-DO-	-DO-	6	None	2
67	406.1	Reactor, Unit 3	135-0	Isolation Valve Comp.	-DO-	2, 3, 15	a) Containment Isolation Valves & Conduits b) CRD insert and Withdrawal Lines	1
68	406.2	-DO-	-DO-	-DO-	-DO-	2 thru 5, 7, 12	-DO-	1

APPENDIX A

SUMMARY OF MASONRY WALLS (SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL NO	WALL NO (I)	WALL DESCRIPTION			WALL'S PRIMARY FUNCTION	SAFETY RELATED SYSTEMS/EQUIPMENT ATTACHED TO WALL (II)	SAFETY RELATED SYSTEMS/EQUIPMENT IN PROXIMITY OF WALL (II)	TENTATIVE RE-EVALUATION PRIORITY
		BUILDING	ELEV.	LOCATION				
69	406.6	Reactor, Unit 3	135-0	Steam Pipe Tunnel	Shielding	None	MSIV on Steam Line "A" and Related Conduits	4
70	406.9	-DO-	-DO-	-DO-	-DO-	5, 12	MSIV on Steam Line "A" and Related Conduits	2
71	406.10	-DO-	-DO-	-DO-	-DO-	3, 5, 10, 12	-DO-	1
72	409.7	-DO-	165-0	Isolation Valve Area	-DO-	None	Containment Isolation Valve & Associated Conduits and RWCU Leakage Detection Temperature Element	4
73	410.1	-DO-	-DO-	Regenerative Heat Exchanger Area	-DO-	5	None	4
74	410.6	-DO-	-DO-	Load Center	Fire Resistance	None	Electrical Load Centers, 7,10,11	2
75	410.7	-DO-	-DO-	-DO-	-DO-	None	-DO-, 2,3,5,6,7,10,11,14,15	1
76	410.8	-DO-	-DO-	-DO-	-DO-	None	-DO-, 10,12	2
77	410.9	-DO-	-DO-	-DO-	-DO-	None	-DO-, 2,3,5,7,10,11,12,14,17	1
78	410.10	-DO-	-DO-	-DO-	-DO-	None	-DO-, 10,11,17	2
79	412.1	-DO-	180-0	Isolation Valve Compartment	Shielding	None	Containment Isolation Valve and Conduits & RWCU Leakage Detection Temp. Element	4
80	418.10	-DO-	234-0	Refueling Floor	-DO-	6	None	4
81	418.11	-DO-	-DO-	-DO-	-DO-	6	-DO-	4
82	532.1	Emergency Cooling Tower	158-0	Emergency Cooling Tower	Fire Resistance	4,8	Emergency Cooling Water Pump and Load Centers and Panels, MCC	4

APPENDIX "A"

SUMMARY OF MASONRY WALLS (SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL NO	WALL NO (I)	WALL DESCRIPTION			WALL'S PRIMARY FUNCTION	SAFETY RELATED SYSTEMS/EQUIPMENT ATTACHED TO WALL (II)	SAFETY RELATED SYSTEMS/EQUIPMENT IN PROXIMITY OF WALL (II)	TENTATIVE RE-EVALUATION PRIORITY
		BUILDING	ELEV.	LOCATION				
83	532.2	Emergency Cooling Tower	158-0	Emergency Cooling Tower	Fire Resistance	8	Load Centers and Panels, MCC	4
84	532.3	-DO-	-DO-	-DO-	-DO-	8	-DO-	4
85	C86.1	Circulating Water Pump Structure	112-0	Pump Room	-DO-	None	Emergency Service Water Piping	4
86	C86.2	-DO-	-DO-	-DO-	-DO-	None	-DO-	4

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APPENDIX "A"

SUMMARY OF MASONRY WALLS (SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2&3

NOTES:

I. Each wall is designated by a unique number such that the first part indicates the Civil/Structural Drawing number and the second part gives the wall number on that particular drawing, e.g. wall No. 45.2 indicates wall number 2 on Civil/Structural Drawing No. 45.

II. Safety related systems are designated as below:

1. Reactor Core Isolation Cooling (RCIC)
2. Reactor Heat Removal (RHR)
3. Core Spray
4. High Pressure Coolant Injection (HPCI)
5. Primary Containment Isolation
6. Process/Area Radiation Monitoring
7. Standby Gas Treatment
8. Emergency [illegible]ce Water (ESW)
9. Emergency D[illegible]el Generator
10. Safeguard 4[illegible] VAC Power (MCC, LC)
11. Safeguard 4 kV Power
12. Safeguard 125/250 VDC Power
13. Safeguard 120 VAC Power
14. Safeguard 24 VDC Power
15. Containment Atmospheric Dilution (CAD)
16. Reactor Protection System (RPS)
17. Safeguard HVAC System
18. Steam Leak Detection System

APPENDIX "B"

SUMMARY OF MASONRY WALL (NON-SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL NO.	WALL NO.	WALL DESCRIPTION			BASIS FOR CLASSIFICATION
		BUILDING	ELEVATION	LOCATION	
1	24.1	Turbine	116-0	Valve Operating Area	Plant Walkdown & Eng. Review
2	32.6	Radwaste	-DO-	Condensate Pump & Tank Rooms	Engineering Review
3	32.7	-DO-	-DO-	-DO-	-DO-
4	32.8	-DO-	-DO-	-DO-	-DO-
5	32.9	-DO-	-DO-	-DO-	-DO-
6	61.1	-DO-	150-0	Hopper Compartment	-DO-
7	61.2	-DO-	-DO-	-DO-	-DO-
8	61.3	-DO-	-DO-	Radwaste DH&V Equipment Compartment	-DO-
9	61.4	-DO-	-DO-	-DO-	-DO-
10	61.5	-DO-	-DO-	-DO-	-DO-
11	61.6	-DO-	-DO-	-DO-	-DO-
12	61.7	-DO-	-DO-	-DO-	-DO-
13	61.8	-DO-	-DO-	-DO-	-DO-
14	78.1	-DO-	165-0	Centrifuge Room	-DO-

APPENDIX "B"

SUMMARY OF MASONRY WALL (NON-SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL NO.	WALL NO.	WALL DESCRIPTION			BASIS FOR CLASSIFICATION
		BUILDING	ELEVATION	LOCATION	
15	78.2	Radwaste	165-0	Centrifuge Room	Engineering Review
16	45.3	Reactor, Unit 2	135-0	Steam Pipe Tunnel	Plant Walkdown & Eng. Rev.
17	45.5	-DO-	-DO-	-DO-	-DO-
18	75.1	-DO-	165-0	Clean Up Recirc. Pump Room	-DO-
19	75.2	-DO-	-DO-	-DO-	-DO-
20	75.3	-DO-	-DO-	-DO-	-DO-
21	75.4	-DO-	-DO-	-DO-	-DO-
22	75.5	-DO-	-DO-	-DO-	-DO-
23	75.7	-DO-	-DO-	-DO-	-DO-
24	76.1	-DO-	-DO-	Regen. Heat Exchanger	-DO-
25	76.2	-DO-	-DO-	Non Regen. Heat Exch. Room	-DO-
26	76.3	-DO-	-DO-	-DO-	-DO-
27	76.4	-DO-	-DO-	Transfer Pump Room	-DO-
28	76.5	-DO-	-DO-	Backwash Rec. Tank Room	Plant Walkdown & Eng. Review

APPENDIX "B"

SUMMARY OF MASONRY WALL (NON-SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL NO.	WALL NO.	WALL DESCRIPTION			BASIS FOR CLASSIFICATION
		BUILDING	ELEVATION	LOCATION	
29	76.11	Reactor, Unit 2	165-0	Transfer Pump Room	Engineering Review
30	97.1	-DO-	195-0	Prefilter & Filter Compartments	Plant Walkdown & Eng. Review
31	97.2	-DO-	-DO-	-DO-	Plant Walkdown
32	97.3	-DO-	-DO-	-DO-	-DO-
33	97.4	-DO-	-DO-	-DO-	-DO
34	101.1	-DO-	234-0	Refueling Floor	-DO-
35	102.1	-DO-	-DO-	-DO-	-DO-
36	102.2	-DO-	-DO-	-DO-	-DO-
37	102.3	-DO-	-DO-	-DO-	-DO-
38	102.4	-DO	-DO-	-DO-	-DO-
39	102.5	-DO-	-DO-	-DO-	-DO-
40	102.6	-DO-	-DO-	-DO-	-DO-
41	102.7	-DO-	-DO-	-DO-	-DO-
42	102.10	-DO-	-DO-	-DO	-DO-

APPENDIX "B"

SUMMARY OF MASONRY WALL (NON-SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL NO.	WALL NO.	WALL DESCRIPTION			BASIS FOR CLASSIFICATION
		BUILDING	ELEVATION	LOCATION	
43	102.11	Reactor, Unit 2	234-0	Refueling Floor	Plant Walkdown
44	406.3	Reactor, Unit 3	135-0	Neutron Monitoring Room	Plant Walkdown & Eng. Rev.
45	406.4	-DO-	-DO-	-DO-	-DO-
46	406.5	-DO-	-DO-	Steam Pipe Tunnel	-DO-
47	406.7	-DO-	-DO-	-DO-	-DO-
48	406.8	-DO-	-DO-	-DO-	-DO-
49	409.1	-DO-	165-0	Clean Up Recirc. Pump Room	-DO-
50	409.2	-DO-	-DO-	-DO-	-DO-
51	409.3	-DO-	-DO-	-DO-	-DO-
52	409.4	-DO-	-DO-	-DO-	-DO-
53	409.5	-DO-	-DO-	-DO-	-DO-
54	409.6	-DO-	-DO-	-DO-	-DO-
55	410.2	-DO-	-DO-	Backwash Rec.Tank Room	Engineering Review
56	410.3	-DO-	-DO-	Transfer Pump Room	-DO-

APPENDIX "B"

SUMMARY OF MASONRY WALL (NON-SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL NO.	WALL NO.	WALL DESCRIPTION			BASIS FOR CLASSIFICATION
		BUILDING	ELEVATION	LOCATION	
57	410.4	Reactor, Unit 3	165-0	Non Regen. Heat Exch. Room	Plant Walkdown & Eng. Rev.
58	410.5	-DO-	-DO-	-DO-	-DO-
59	410.11	-DO-	-DO-	Transfer Pump Room	-DO-
60	413.1	-DO-	195-0	Storage Area	Plant Walkdown
61	413.2	-DO-	-DO-	-DO-	-DO-
62	413.3	-DO-	-DO-	-DO-	-DO-
63	413.4	-DO-	-DO-	-DO-	-DO-
64	413.5	-DO-	-DO-	-DO-	-DO-
65	413.6	-DO-	-DO-	-DO-	-DO-
66	414.1	-DO-	-DO-	Prefilter & Filter Compartment	Plant Walkdown & Engr. Review
67	414.2	-DO-	-DO-	-DO-	Plant Walkdown
[illegible]	414.3	-DO-	-DO-	-DO-	-DO-
[illegible]	414.4	-DO-	-DO-	-DO-	-DO-
[illegible]0	417.1	-DO-	234-0	Refueling Area	-DO-

APPENDIX "B"

SUMMARY OF MASONRY WALL (NON-SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL NO.	WALL NO.	WALL DESCRIPTION			BASIS FOR CLASSIFICATION
		BUILDING	ELEVATION	LOCATION	
71	418.1	Reactor, Unit 3	234-0	Refueling Area	Plant Walkdown
72	418.2	-DO-	-DO-	-DO-	-DO-
73	418.3	-DO-	-DO-	-DO-	-DO-
74	418.4	-DO-	-DO-	-DO-	-DO-
75	418.5	-DO-	-DO-	-DO-	-DO-
76	418.6	-DO-	-DO-	-DO-	-DO-
77	418.7	-DO-	-DO-	-DO-	-DO-
78	418.8	-DO-	-DO-	-DO-	-DO-
79	418.9	-DO-	-DO-	-DO-	-DO-
80	418.12	-DO-	-DO-	-DO-	-DO-
81	418.13	-DO-	-DO-	-DO-	-DO-
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* NOTE: Based on Engineering review, all masonry walls of Recombiner Building and masonry walls of Radwaste, Turbine, and Reactor Building (Units 2 & 3) not listed above, are non-safety related.

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