PHILADELPHIA ELECTRIC COMPANY

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July 2, 1980

Docket Nos.: 50-277 50-278

IE Bulletin 80-11

Mr. Boyce H. Grier, Director Office of Inspection & Enforcement Region I U.S. Nuclear Regulatory Commission 631 Park Avenue King of Prussia, PA 19406

Dear Mr. Grier:

This is in response to your letter of May 8, 1980, which forwarded IE Bulletin 80-11, and requires a response to items 1, 2a, and 3 of the Bulletin within 60 days of the issue date. A response to item 2b of the Bulletin will be forwarded within 180 days of the issue date of the Bulletin. The actions requested in items 1, 2a, and 3 of the Bulletin, and our responses are listed sequentially below.

Action to be Taken by Licensee:

1. Identify all masonry walls in your facility which are in proximity to or have attachments from safety-related piping or equipment such that wall failure could affect a safety-related system. Describe the systems and equipment, both safety and non-safety-related, associated with these masonry walls. Include in your review, masonry walls that are intended to resist impact or pressurization loads, such as missiles, pipe whip, pipe break, jet impingement, or tornado, and fire or water barriers, or shield walls. Equipment to be considered as attachments or in proximity to the walls shall include, but is not limited to, pumps, valves, motors, heat exchangers, cable trays, cable/conduit, HVAC ductwork, and electrical cabinets, instrumentation and controls. Plant surveys, if necessary, for areas inaccessible during normal

plant operation shall be performed at the earliest opportunity.

Response

All masonry walls as Peach Bottom Atomic Power Station which are in proximity to or nave attachments from safety-related piping or equipment are identified in Appendix A. Proximity is defined as being within a distance equal to the wall height, such that wall failure could render the safety-related system inoperable. Safety-related systems and equipment associated with these masonry walls are also described in Appendix A, together with the wall location, its primary function (shield wall, partition, etc.), and the tentative priority for re-evaluation.

Non-safety-related systems or equipment such as conduits, cable trays etc. attached to these walls have been surveyed and all data related to these systems have been obtained in order to establish imposed loading on masonry walls for use during the reevaluation analysis. The information shown in Appendix A was obtained by performing a plant survey.

Appendix B lists masonry walls which do not have safety-related systems/equipment either attached to them or in proximity. The information shown in Appendix B was obtained either by plant survey and/or by drawing review.

A visual, walk-through verification will be made to confirm the drawing review. Should this investigation identify any walls which affect safety-related systems or equipment, those walls will be surveyed in detail, added to Appendix A, and included in the re-evaluation program.

- 2. Provide a re-evaluation of the design adequacy of the walls identified in Item 1 above to determine whether the masonry walls will perform their intended function under all postulated loads and load combinations. In this regard, the NRC encourages the formation of an owners' group to establish both appropriate re-evaluation criteria and where necessary, a later confirmatory masonry test program to quantify the safety margins established by the re-evaluation criteria (this is discussed further in Item 3 below).
 - a. Establish a prioritized program for the re-evaluation of the masonry walls. Provide a description of the program and a detailed schedule for completion of the re-evaluation for the categories in the program. The completion date of all re-evaluations should not be more than 180 days from the date of this Bulletin. A higher priority should be placed on the wall re-evaluations considering safety-related piping 2-1/2 inches or

greater in diameter, piping with support loads due to thermal expansion greater than 100 pounds, safety-related equipment weighing 100 pounds or greater, the safety significance of the potentially affected systems, the overall loads on the wall, and the opportunity for performing plant surveys and, if necessary, modifications in areas otherwise inaccessible. The factors described above are meant to provide guidance in determining what loads may significantly affect the masonry wall analyses.

Response

The re-evaluation program will consist of a structural analysis of the masonry walls identified in Appendix A using an appropriate re-evaluation criteria which will define loads considered, load combinations and allowable stresses.

The following parameters will be considered in establishing priorities for wall re-evaluation:

- a) Walls which have larger number of safety-related systems on them or in their proximity.
- b) Walls which are subjected to heavier loading, and
- c) Walls which, in terms of height, length and thickness are judged to be more critical than others.

Tentative re-evaluation priorities are indicated for each wall in Appendix A. Priority 1 and Priority 2 indicate the walls which will be analyzed first, followed by Priority 3 and 4 in that order.

The re-evaluation of masonry walls designated as Priority 1 and Priority 2 is expected to be completed by mid or late September, 1980. Re-evaluation of Priority 3 and Priority 4 masonry walls is expected to be completed by mid or late October, 1980. A report of the re-evaluation will be submitted by November 4, 1980, as required by the Bulletin.

3. Existing test data or conservative assumptions may be used to justify the re-evaluation acceptance criteria if the criteria are shown to be conservative and applicable for the actual plant conditions. In the absence of appropriate acceptance criteria a confirmatory masonry wall test program is required by the NRC in order to quantify the safety margins inherent in the re-evaluation criteria. Describe in detail the actions planned and their schedule to justify the re-evaluation criteria used in Item 2. If a test program is necessary, provide your commitment for such a program and a

and a schedule for completion of the program. This test program should address all appropriate loads (seismic, tornado, missile, etc.). It is expected that the test program will extend beyond the 180 day period allowed for the other Bulietin actions. Submit the results of the test program upon its completion.

Response

Justification for re-evaluation criteria will be submitted with the re-evaluation report within 180 days of the date of the Bulletin. Justification will be based on reference to effective codes and established standards of practice related to concrete and masonry design typically used throughout the industry.

It is anticipated that such justification will be considered appropriate, except as required to determine project unique structural properties such as collar joint strength and any other properties for which test data is not available or cannot otherwise be determined. Should these unique properties need to be determined by testing, adequate test procedures will be developed as required.

Very truly yours,

Alfatty (

Attachments:

Appendix A Appendix B

cc: United States Nuclear Regulatory Commission Office of Inspection and Enforcement Division of Reactor Operations Inspection Washington, DC 20555 COMMONWEALTH OF PENNSYLVANIA :

ss.

COUNTY OF PHILADELPHIA

S. L. Daltroff, being first duly sworn, deposes and says:

That he is Vice President of Philadelphia Electric Company; that he has read the foregoing response to IE Bulletin 80-11 and knows the contents thereof; and that the statements and matters set forth therein are true and correct to the best of his knowledge, information and belief.

Subscribed and sworn to

before me this 300 day

of July, 1980

EUZABETH H. BOVER

Notary Public, Phila., Phila. Co. Ay Commission Expires Jan. 30, 1982

Sheet 1 of 7

APPENDIX "A"

SUMMARY OF MASONRY WALLS (SAFETY PELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL	WALL		WALL DESK	RIPTION	WALL'S PRIMURY	SAPETY RELATED SYSTEMS/EQUIPMENT		TOVITATIVE
NO	NO (1)	BUILDING	ELEV.	LOCATION	FUNCTION	ATTACRES TO WALL (II)	PROXIDITY OF WALL (II)	RE-EVALUATION PRIORITY
1	15.1	Radwaste	88-0 & 91-6	Standby Gas Treatment Room	Shielding	7	Standby Gas Treatment System (Fans, Filters and Ducts)	3
2	15.2	-DO-	-DO-	-DO-	-DO-	7	-10-	3
3	16.1	-10-	-DO-	Reactor Core Isolation Cooling Pump Area, Unit 2	-DO-	None	Reactor Core Isolation Cooling Pump Turbine, Unit 2	3
4	16.2	-00-	-DO-	-DO-, Unit 3	-DO-	None	-DO-, Unit 3	3
5	16.3	-DO-	-00-	Waste Sludge Tank Room	-DO-	None	Safeguard Related Conduit	3
6	32.1	-DO-	116-0	Condensate Tank & Pump Rooms	-DO-	7, 10	None	3
7	32.2	-DO-	-DO-	-DO-	-DO-	None	480V AC Power	3
8	32.3	-00-	-DO-	Condensats Pump Area	-DO-	1, 7	SGT & RCEC	3
9	32.4	-DO-	-DO-	-00-	-DO-	-DO-	-DO-	3 .
10	32.5	-DO-	-DO-	-D0-	-DO-	-DO-	-DO-	3
11	32.10	-D0-	-DO-	Filter Holding Pump Room & Fuel Pool Filter Demineralizer Area	Shielding	7	None	3
12	32.11	-DO-	-DO-	-D0-	-DO-	None	1	3
13	32.12	-DO-	-00-	-10-	-DO-	None	.7.	3
14	56.1	-10-	135-0	RW Control Room & Filter Aid Tank & Pump Area	-DO-	6	Mone	3
15	78.3	-DO-	165-0	Redweste Building "	-DO-	17	Energ. Swgr Filter Mousing	3
16	25.1	Turbine (Con- trol Structure)	116-0	Switchgear & Conventional Chem. Lab	Partition	2, 3, 8 thru'll	Righ Pressure and Energency Service Water Lines and Safeguard Conduits	2
17	25.2	-10-	-DO-	-DO-	-DO-	2, 3		5
18	40.1	-DO-	135-0	Battery Room, Unit 2	F re Resistance	8, 10, 12, 14	Battery Rack, Channels A & C, Unit 2	1

APPENDIX"A*

BERIAL	WALL L		WALL DESK	RIPTION	WALL'S PRIMARY	SAFETY RELATED SYSTEMS/EQUIPMENT	SAFETY RELATED SYSTEMS/BOULDHENT IN	TENTA:IVE
NO	NO (I)	BUILDING	ELEV.	LOCATION	FUNCTION	ATTACHED TO WALL (II) ;	PROXIMITY OF WALL (II)	PRIORITY
19	40.2	Turbine (Control Structure)	135-0	Switchgear Room Unit 2	Fire Resistance	2, 7, 9, 10, 12, 13, 14, 17	4 kV Switchgear, Channel C, & MCC Channel C, Unit 2	1
50	40.3	- 00-	-DO-	-DO-	-DO-	1, 2, 3, 6 thru' 14, 17.	4 kV Switchgear, Channel D, Unit 2	1
21	40.4	-00-	-DO-	-DO-, Unit 3	-00-	2, 8, 9, 11, 12,13, 17	-DC-, Channel D, Unit 3	1
55	40.5	-DO-	-DO-	-DO-, Unit 3	-DO-	2, 3, 5, 6, 7, 9, 10, 12, 13,	-DO-, Channel C, Unit 3	1
23	40.6	-00-	-DO-	Battery Room, Unit 3	-DO-	1, 8, 10, 12, 14	Battery Rack, Channels A & C, Unit 3	1
24	40.7	-00-	-00-	Battery Room, Unit 2	-DO-	10, 12, 14	Battery Rack, Channels A, B, C, & D, Unit 2 and Puse Boxes Channels A&C.	1
25	40.8	-DO-	-DO-	Switchgear Room, Unit 2	-00-	7, 10, 12, 14	4 kV Switchgear, Channels A & C, Unit 2	1
26	40.9	-DO-	-DO-	-DO-, Unit 2	-DO-	7, 9 thru'13	-DO-, Channels B & D, Unit 2	1
27	40.10	-100-	-DO-	-DO-, Unit 3	-10-	8, 9, 10, 17	-DO-, Channels B & D, Unit 3	1
28	40.11	-DO-	-DO-	-DO-, Unit 3	-DO-	2, 3, 7 thru'12, 14, 17	-DO-, Channels A & C, Unit 3	1
29	40.12	-00-	-DO-	Battery Room, Unit 3	-00-	1, 8, 10, 12	Battery Rack, Channels A, B, C, & D, Unit 3 and Fuse Boxes Channels A&C	1
30	40.13	-00-	-DO-	Battery Room & Switch- gear, Unit 2	-10-	6, 8 thru'12, 14, 17	Battery Rack, Channels A & C, MCC Channel C, Fuse Boxes Channels, A & C 4 kV Switchgear, Channel C, Unit 2	1
31	40.14	- 00-	-DO-	-DO-	-10-	2, 8, 10, 11, 12, 14	Battery Rack, Channels B & D, MCC Channel A, Fuse Boxes Channels B & D. 4 kV Switchgear, Channel A, Unit 2	1
32	40.15	-00-	-DO-	Switchgear Room, Unit 2	-10-	2, 9, 11, 12	4 kV Switchgear, Channels C & D, Unit 2	1
33	40.16	- 00-	-DO-	Switchgear Room, Unit 2	-DO-	2, 8 thru 14	4 kV Switchgear, Channels A & B, Channel B, Unit 2	1 1
34	40.17	-10-	-DO-	Switchgear Room, Units 2	-10-	2, 8 thru'13	4 kV Switchgear, Channels D & D, Units 2 & 3	1 .

RIAL	WALL NO (I)		WALL D	ESCRIPTION	Laurin	WALL'S PRINGRY SAFETY NELATED SYSTEMS AND TRANSPORT		
	mo (1)	BUILDING	ELEV.	LOCATION	PUNCTION PUNCTION	SAFETY RELATED SYSTEMS/EQUIPMENT ATTACHED TO WALL (II)	SAVETY RELATED SYSTEMS/BULLPHENT IN PROXIMITY OF WALL (II)	TENTATIVE
35	40.18	Turbine (Control Structure)	135-0	Switchgear Room, Units 2	Fire Resistance	2, 9, 11, 12	4 kV Switchgear, Channels B & B, Units 2	PRIORITY
6	40.19	-DO-	-DO-	Switchgear Room, Unit 3	-00-		4 3,	1
7	40.20	-00-	-DO-	-10-	-DO-	8, 9, 11, 12, 17	-DO-, Channels C & D, MCC Channel D,	1
3	40.21	-DO-	-DO-	Switchgear & Battery	-00-	2, 7, 8, 9, 11 thru' 14	-DO-, Channels A & B, MCC Channel B,	1
1	40.22	-00-	-00-	Rooms, Unit 3			Battery Rack, Channels A & C, Tuse Boxes	1
	40.23			-10-	-DO-	1	k kV Switchgear, Channel C, Unit 3 Battery Rack, Channels B & D, MCC	
	40.23	-DO-	-DO-	Switchgear Room, Unit 2	-00-		kV Switchgear, Channel A, Unit 3	1
1	40.24	-DO-	-DO-	-DO-, Unit 2	-DO-		4 kV Switchgear, Channel D, Unit 2 and	3
1	40.25	-DO-	-DO-	-DO-, Unit 3	-00-	None	-00-	3
1	40.26	-DO-	-DO-	-DO-, Unit 3	-DO-	Mone	-DO-, Unit 3	3
1	40.28	-DO-	-DO-	-DO-, Unit 2	-DC	None	-DO-	3
1	10.29	-DO-	-DO-	-DO-	-DO-	None	4 kV Switchgear & MCC Channel A, Unit 2	3
	0.30	-DO-	-DO-	-DO-, Unit 3	-DO-	None	kV Switchgear & MCC Channel B, Unit 2	3
	8.1	-DO-	-DO-	-DO-	-10-		kV Switchgear & MCC Channel B, Unit 3	3
	8.2	-DO-		Cable Spreading Room	-DO-	1 8 0 11 12 12	kV Switchgear & MCC Channel A, Unit 3	3
6	3.3	-DO-	-DO-	-DO-	-DO-	1	-10-	
68	1.4	-DO-	-DO-	-20-	-DO-	1 thru'5, 7 thry'12, 14, 18	-00-	2
68	.5	-DO-	-DO-	-m		1 thru'.7, 9 thru'.17	-10-	1
					Shielding	None	-DO-	2

APPENDIX "A"

SUMMARY OF MASONRY WALLS (SAFETY RELATED)

SERIAL	WALL		WALL DES	SCRIPTION	WALL'S PRIMIRY	SAFETY RELATED SYSTEMS/EQUIPMENT	I mant temporar profit by agont that the	TENTATIVE
NO	NO (1)	BUILDING	ELEV.		FUNCTION	ATTACHED TO WALL (II)	PHOXIMITY OF WALL (II)	PRIORITY
53	71.1	Turbine (Control Structure)	165-0	Control Room	Shielding	None	Control Panels, Safeguard Trays	2
54	45.1	. Reactor, Unit 2	135-0	Isolation Valve Comp.	-DO-	2, 3, 5	a) Containment Isolation Valves and Assoc. Conduits b) CRD Insert and Withdrawal Lines	1
55	15.2	-DO-	-DO-	-00-	-DO-	2, 3, 4, 10, 15	-D0-	1
56	45.4	-00-	-DO-	Steam Pipe Tunnel	-10-	5, 15, 18	MSIV on Steam Line "A" and Related Conduits	2
57						None	5, 15, 18	2
	45.6	-700-	-00-	-D0-	-00-			
58	75.6	-00-	165-0	Isolation Valve Area	-DO-	None	& RNCU leakage detection temp. element	,
59	76.6	-DO-	-DO-	Load Centers	Fire Resistance	9, 10	Electrical Load Center	. 4
60	76.7	-00-	-DO-	-00-	-D0-	None	-DO-, 1 thru 7, 9, 10, 15, 17	1
61	76.8	-DO-	-DO-	-10-	-00-	None	Electrical Load Center	
62	76.9	-D0-	-D0-	-DO-	-DO-	None	-DO-, 1 thru 7, 9, 10, 15	1 '
63	76.10	-DO-	-DO-	-00-	-DO-	2	Electrical Load Center	
64	87.1	-DO-	180-0	Isolation Valva Compartment	Shielding	None	Containment Isolation Valves, RWCU	
STACE			1					
65	102.8	-10-	234-0	Refueling	-10-	6	None	
66	102.9	-00-	-DO-	-DO-	-DO-	6	None	2
67	406.1	Reactor, Unit 3	135-0	Isolation Valve Comp.	-DO-	2, 3, 15	a) Containment Isolation Valves & Conduits b) CRD insert and Withdrawal Lines	1
68	406.2	-DO-	-DO-	-D0-	-DO-	2 thru'5, 7, 12	-10-	1

APPENDIX A

SUMMARY OF MASONRY WALLS (SAFETY RELATED)

SERIAL	WALL		WALL DES	CRIPTION	WALL'S PRIMURY	SAFETY RELATED SYSTEMS/EQUIPMENT	SAFETY RELATED SYSTEMS/EQUIPMENT IN	TENTATIVE
NO	NO (I)	BUILDING	ELEV.	LOCATION	FUNCTION	ATTACHED TO WALL (II)	PHOKIMITY OF WALL (II)	PRIORITY
69	406.6	Reactor, Unit 3	135-0	Steam Pipe Tunnel	Shielding	None	MSIV on Steam Line "A" and Related Conduits	
70	406.9	-00-	-DO-	-DO-	-DO-	5, 12	MSIV on Steam Line "A" and Related Conduits	2
71	406.10	-DO-	-DO-	-10-	-DO-	3, 5, 10, 12	-100-	1
72	409.7	-DO-	165-0	Isolation Valve Area	-D0-	None	Containment Isolation Valve & Associated Conduits and RWCU Leakage Detection Temperature Element	
73	410.1	-DO-	-DO-	Regenerative Heat Ex- changer Area	-DO-	,	None	•
74	410.6	-00-	-DO-	Load Center	Fire Resistance	None	Electrical Load Centers, 7,10,11	2
75	410.7	-00-	-DO-	-00-	-DO-	None	-10-, 2,3,5,6,7,10,11,14,15	1
76	410.8	-DO-	-D0-	-DO-	-00-	None	-00-, 10,12	2
77	410.9	-00-	-DO-	-DO-	-DO-	None	-00-, 2,3,5,7,10,11,12,14,17	1
78	410.10	-DO-	-00-	-DO-	-DO-	None	-10-, 10,11,17	2
79	412.1	-00-	180-0	Isolation Valve Compartment	Shielding	None	Containment Isolation Valve and Conduits & RWCU Leakage Detection Tomp. Element	
80	418.10	-DO-	231-0	Refueling Floor	-00-	6	None	4
81	418.11	-00-	-DO-	-00-	-DO-	6	-00-	4
82	532.1	Emergency Cooling Tower	158-0	Emergency Cooling Tower	Fire Resistance	4,8	Emergency Cooling Water Pump and Load Centers and Panels, MCC	4

SUMMARY OF MASONRY WALLS (SAFETY RELATED)
PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

TEMPATIVE	PRIORITY		•	•	•	
SAFETY RELATED SISTING REQUIRENT IN	PROGNETY OF WALL (11)	Load Centers and Panels, MCC	ė.	Brargency Service Water Piping	-00-	
SAFET RELATED SYSTEMS/BOUTHERN	ATTOO BY TO WALL (II)		8	None	None	
WALL'S PRINGJEY	FUNCTION	Fire Resistance	-00-	97	-bd-	
BIFTIGN	ELEV. LOCATION	Emergency Couling Tower	-02-	Pump Roces	-10,	
SEE IES	EEV.	158-0	94	112-0	ģ	
	BUILDING	Exergency Cooling Tower	-80-	Circulating Water Pump Structure	-00-	
MALL	NO (1)	532.2	532.3	086.1	2.985.2	
SERIAL	0.1	93	18	85	98	

APPENDIX "A"

SUMMARY OF MASONRY WALLS (SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 243

NOTES:

I. Each wall is designated by a unique number such that the first part indicates the Civil/Structural Drawing number and the second part gives the wall number on that particular drawing, e.g. wall No. 45.2 indicates wall number 2 on Civil/Structural Drawing No. 45.

II. Safety related systems are designated as below:

- 1. Reactor Core Isolation Cooling (RCIC)
- 2. Reactor Heat Removal (RHR)
- 3. Core Spray
- 4. High Pressure Coolant Injection (HPCI)
- 5. Primary Containment Isolation
- 6. Process/Area Radiation Monitoring
- 7. Standby Gas Treatment
- 8. Emergency Service Water (ESW)
- 9. Emergency Diesel Generator
- 10. Safeguard 480 VAC Power (MCC, LC)
- 11. Safeguard 4 kV Power
- 12. Safeguard 125/250 VDC Power
- 13. Safeguard 120 VAC Power
- 14. Safeguard 24 VDC Power
- 15. Containment Atmospheric Dilution (CAD)
- 16. Reactor Protection System (RPS)
- 17. Safeguard HVAC System
- 18. Steam Leak Detection System

APPENDIX "B"

SUMMARY OF MASONRY WALL (NON-SAFETY RELATED) PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

ERIAL				DESCRIPTION			
NO.	WALL NO.	BUILDING	ELEVATION	LOCATION	BASIS FOR CLASSIFICATION		
1	24.1	Turbine	116-0	Valve Operating Area	Plant Walkdown & Eng. Revie		
2	32.6	Radwaste	-DO-	Condensate Pump & Tank Rooms	Engineering Review		
3	32.7	-DO-	-DO-	-DO-	-DO-		
4	32.8	-DO-	-DO-	-DO-	-DO-		
5	32.9	-DO-	-DO-	-DO-	-DO-		
6	61.1	-DO-	150-0	Hopper Compartment	-DO-		
7	61.2	-DO-	-DO-	-DO-	-DO-		
8	61.3	-DO-	-DO-	Radwaste DH&V Equipment Compartment	-DO-		
9	61.4	-DO-	-DO-	-DO-	-DO-		
.0	61.5	-DO-	-DO-	-DO-	-DO-		
1	61.6	-DO-	-DO-	-DO-	-DO-		
12	61.7	-DO-	-DO-	-DO-	-DO-		
13	61.8	-DO-	-DO-	-DO-	-DO-		
14	78.1	-DO-	165-0	Centrifuge Room	-DO-		
	i i						

APPENDIX "B"

SERIA	L	Test test	WALL D	ESCRIPTION	
NO.	WALL NO.	BUILDING	ELEVATION	LCCATION	BASIS FOR CLASSIFICATION
15	78.2	Radwaste	165-0	Centrifuge Room	Engineering Review
16	45.3	Reactor, Unit 2	135-0	Steam Pipe Tunnel	Plant Walkdown & Eng. Rev.
17	45.5	-DO-	-DO-	-DO-	-DO-
18	75.1	-DO-	165-0	Clean Up Recirc. Pump Room	DO-
19	75.2	-DO-	-DO-	-DO-	-DO-
20	75.3	-DO-	-DO-	-DO-	-DO-
21	75.4	-DO-	-DO-	-DO-	-DO-
22	75.5	-DO-	-DO-	-DO-	-DO-
23	75.7	-DO-	-DO-	-DO-	-DO-
24	76.1	-DO-	-DO-	Regen. Heat Exchanger	-DO-
25	76.2	-DO-	-DO-	Non Regen. Heat Exch. Room	-DO-
26	76.3	-DO-	-DO-	-DO-	DO
27	76.4	-DO-	-DO-	Transfer Pump Room	-DO-
28	76.5	-DO-	-DO-	Backwash Rec. Tank Room	Plant Walkdown & Eng. Review

APPENDIX "B"

SUMMARY OF MASONRY WALL (NON-SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIA	L			SCRIPTION	PAGES BOD CLASSIFICATION
NO.	WALL NO. 1	BUILDING	ELEVATION	LOCATION	BASIS FOR CLASSIFICATION
29	76.11	Reactor, Unit 2	165-0	Transfer Pump Room	Engineering Review
30	97.1	-DO-	195-0	Prefilter & Filter Compartments	Plant Walkdown & Eng. Revie
31	97.2	-DO-	-DO-	-DO-	Plant Walkdown
32	97.3	-DO-	-DO-	-DO-	-DO-
33	97.4	-DO-	-DO-	-DO-	-DO
34	101.1	-DO-	234-0	Refueling Floor	-DO-
35	102.1	-DO-	-DO-	-DO-	-DO-
36	102.2	-DO-	-DO-	-DO-	-DO-
37	1102.3	-DO-	-DO-	-DO-	-DO-
38	1102.4	-DO	-DO-	-DO-	-DO-
39	1102.5	-DO-	-DO-	-DO-	-DO-
40	102.6	-DO-	-DO-	-DO-	-DO-
41	1102.7	DO-	-DO-	-DO-	-DO-
42	1102.10	-DO-	-DO-	_DO	-DO-

APPENDIX "B"

SERIAL				ESCRIPTION	
NO.	WALL NO.	BUILDING	ELEVATION	LOCATION	BASIS FOR CLASSIFICATION
43	102.11	Reactor, Unit 2	234-0	Refueling Floor	Plant Walkdown
44	406.3	Reactor, Unit 3	135-0	Neutron Monitoring Room	Plant Walkdown & Eng. Rev.
45	406.4	-DO-	-DO-	-DO-	-DO-
46	406.5	-DO-	-DO-	Steam Pipe Tunnel	-DO-
47	406.7	-DO-	-DO-	-DO-	-DO-
48	406.8	-DO-	-DO-	-DO-	-DO-
49	409.1	-DO-	165-0	Clean Up Recirc. Pump Room	-DO-
50	1409.2	-DO-	-DO-	-DO-	-DO
51	1409.3	-DO-	-DO-	-DO-	-DO-
52	409.4	-DO-	-DO-	-DO-	-DO-
53	1409.5	-DO-	-DO-	-DO-	-DO-
54	409.6	-DO-	-DO-	-DO-	-DO-
55	410.2	-DO-	-DO-	Backwash Rec.Tank Room	Engineering Review
56	1410.3	-DO-	-DO-	Transfer Pump Room	-DO-

APPENDIX "B"

SERIA	L		WALL D	ESCRIPTION	4 - A - C - C - C - C - C - C - C - C - C
NO.	WALL NO.	BUILDING	ELEVATION	LOCATION	BASIS FOR CLASSIFICATION
57	410.4	Reactor, Unit 3	165-0	Non Regen. Heat Exch. Room	Plant Walkdown & Eng. Rev.
58	410.5	-DO-	-DO-	-DO-	-DO-
59	410.11	-DO-	-DO-	Transfer Pump Room	-DO-
60	413.1	-DO-	195-0	Storage Area	Plant Walkdown
61	413.2	-DO-	-DO-	-DO-	-DO-
62	413.3	-DO-	-DO-	-DO-	-DO-
63	413.4	-DO-	-00-	-DO-	-DO-
64	413.5	-DO-	-DO-	-DO-	-DO-
65	413.6	-DO-	-DO-	-DO-	-DO-
66	414.).	-DO-	-DO	Prefilter & Filter Compartment	Plant Walkdown & Engr. Review
67	414.2	-DO-	-DO-	-DO-	Plant Walkdown
68	1414.3	-DO-	-DO-	-DO-	-DO-
67	414.4	-DO-	-DO-	-DO-	-DO-
70	417.1	-DO-	234-0	Refueling Area	-DO-

APPENDIX "B"

SERIAL			WALL DES	CRIPTION	
NO.	WALL NO. 1	BUILDING	ELEVATION	LOCATION	BASIS FOR CLASSIFICATION
71	418.1	Reactor, Unit 3	234-0	Refueling Area	Plant Walkdown
72	1418.2	-DO-	-DO-	-DO-	-DO-
73	1418.3	-DO-	-DO-	-DO-	-DO-
74	418.4	-DO-	-DO-	-DO-	-DO-
75	1418.5	-DO-	-DO-	-DO-	-DO-
76	1418.6	-DO-	-DO-	-DO-	-DO-
77	1418.7	-DO-	-DO-	-DO-	-DO-
78	1418.8	-DO-	-DO-	-DO-	-DO-
79	1418.9	-DO-	-DO-	-DO-	-DO-
80	418.12	-DO-	-DO-	-DO-	-DO-
81	1418.13	-DO-	-DO-	-DO-	-DO-

^{*} NOTE: Based on Engineering review, all masonry walls of Recombiner Building and masonry walls of Radwaste, Turbine, and Reactor Building (Units 2 & 3) not listed above, are non-safety related.

CENTIN FLES PHILADELPHIA ELECTRIC COMPANY 2301 MARKET STREET P.O. BOX 8699 PHILADELPHIA, PA. 19101 (215) 841-5001 SHIELDS L. DALTROFF VICE PRESIDENT July 2, 1980 Docket Nos.: 50-277 50-278 IE Bulletin 80-11 Mr. Boyce H. Grier, Director Office of Inspection & Enforcement Region I U.S. Nuclear Regulatory Commission 631 Park Avenue King of Prussia, PA 19406 Dear Mr. Grier: This is in response to your letter of May 8, 1980, which

This is in response to your letter of May 8, 1980, which forwarded IE Bulletin 80-11, and requires a response to items 1, 2a, and 3 of the Bulletin within 60 days of the issue date. A response to item 2b of the Bulletin will be forwarded within 180 days of the issue date of the Bulletin. The actions requested in items 1, 2a, and 3 of the Bulletin, and our responses are listed sequentially below.

Action to be Taken by Licensee:

1. Identify all masonry walls in your facility which are in proximity to or have attachments from safety-related piping or equipment such that wall failure could affect a safety-related system. Describe the systems and equipment, both safety and non-safety-related, associated with these masonry walls. Include in your review, masonry walls that are intended to resist impact or pressurization loads, such as missiles, pipe whip, pipe break, jet impingement, or tornado, and fire or water barriers, or shield walls. Equipment to be considered as attachments or in proximity to the walls shall include, but is not limited to, pumps, valves, motors, heat exchangers, cable trays, cable/conduit, HVAC ductwork, and electrical cabinets, instrumentation and controls. Plant surveys, if necessary, for areas inaccessible during normal

plant operation shall be performed at the earliest opportunity.

Response

All masonry walls at Peach Bottom Atomic Power Station which are in proximity to or have attachments from safety-related piping or equipment are identified in Appendix A. Proximity is defined as being within a distance equal to the wall height, such that wall failure could render the safety-related system inoperable. Safety-related systems and equipment associated with these masonry walls are also described in Appendix A, together with the wall location, its primary function (shield wall, partition, etc.), and the tentative priority for re-evaluation.

Non-safety-related systems or equipment such as conduits, cable trays etc. attached to these walls have been surveyed and all data related to these systems have been obtained in order to establish imposed loading on masonry walls for use during the re-evaluation analysis. The information shown in Appendix A was obtained by performing a plant survey.

Appendix B lists masonry walls which do not have safety-related systems/equipment either attached to them or in proximity. The information shown in Appendix B was obtained either by plant survey and/or by drawing review.

A visual, walk-through verification will be made to confirm the drawing review. Should this investigation identify any walls which affect safety-related systems or equipment, those walls will be surveyed in detail, added to Appendix A, and included in the re-evaluation program.

- 2. Provide a re-evaluation of the design adequacy of the walls identified in Item 1 above to determine whether the masonry walls will perform their intended function under all postulated loads and load combinations. In this regard, the NRC encourages the formation of an owners' group to establish both appropriate re-evaluation criteria and where necessary, a later confirmatory masonry test program to quantify the safety margins established by the re-evaluation criteria (this is discussed further in Item 3 below).
 - a. Establish a prioritized program for the re-evaluation of the masonry walls. Provide a description of the program and a detailed schedule for completion of the re-evaluation for the categories in the program. The completion date of all re-evaluations should not be more than 180 days from the date of this Bulletin. A higher priority should be placed on the wall re-evaluations considering safety-related piping 2-1/2 inches or

greater in diameter, piping with support loads due to thermal expansion greater than 100 pounds, safety-related equipment weighing 100 pounds or greater, the safety significance of the potentially affected systems, the overall loads on the wall, and the opportunity for performing plant surveys and, if necessary, modifications in areas otherwise inaccessible. The factors described above are meant to provide guidance in determining what loads may significantly affect the masonry wall analyses.

Response

The re-evaluation program will consist of a structural analysis of the masonry walls identified in Appendix A using an appropriate re-evaluation criteria which will define loads considered, load combinations and allowable stresses.

The following parameters will be considered in establishing priorities for wall re-evaluation:

- a) Walls which have larger number of safety-related systems on them or in their proximity.
- b) Walls which are subjected to heavier loading, and
- c) Walls which, in terms of height, length and thickness are judged to be more critical than others.

Tentative re-evaluation priorities are indicated for each wall in Appendix A. Priority 1 and Priority 2 indicate the walls which will be analyzed first, followed by Priority 3 and 4 in that order.

The re-evaluation of masonry walls designated as Priority 1 and Priority 2 is expected to be completed by mid or late September, 1980. Re-evaluation of Priority 3 and Priority 4 masonry walls is expected to be completed by mid or late October, 1980. A report of the re-evaluation will be submitted by November 4, 1930, as required by the Bulletin.

3. Existing test data or conservative assumptions may be used to justify the re-evaluation acceptance criteria if the criteria are shown to be conservative and applicable for the actual plant conditions. In the absence of appropriate acceptance criteria a confirmatory masonry wall test program is required by the NRC in order to quantify the safety margins inherent in the re-evaluation criteria. Describe in detail the actions planned and their schedule to justify the re-evaluation criteria used in Item 2. If a test program is necessary, provide your commitment for such a program and a

schedule for submittal of a description of the test program and a schedule for completion of the program. This test program should address all appropriate loads (seismic, tornado, missile, etc.). It is expected that the test program will extend beyond the 180 day period allowed for the other Bulletin actions. Submit the results of the test program upon its completion.

Response

Justification for re-evaluation criteria will be submitted with the re-evaluation report within 180 days of the date of the Bulletin. Justification will be based on reference to effective codes and established standards of practice related to concrete and masonry design typically used throughout the industry.

It is anticipated that such justification will be considered appropriate, except as required to determine project unique structural properties such as collar joint strength and any other properties for which test data is not available or cannot otherwise be determined. Should these unique properties need to be determined by testing, adequate test procedures will be developed as required.

Very truly yours,

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Attachments:

Appendix A Appendix B

cc: United States Nuclear Regulatory Commission Office of Inspection and Enforcement Pivision of Reactor Operations Inspection Ishington, DC 20555 COMMONWEALTH OF PENNSYLVANIA :

ss.

COUNTY OF PHILADELPHIA

S. L. Daltroff, being first duly sworn, deposes and says:

That he is Vice President of Philadelphia Electric Company; that he has read the foregoing response to IE Bulletin 80-11 and knows the contents thereof; and that the statements and matters set forth therein are true and correct to the best of his knowledge, information and belief.

Subscribed and sworn to

before me this 312 day

of July, 198

ELIZABETH H. BOYER

Notary Public, Phila., Phila. Co. My Commission Expires Jan. 30, 1982

APPENDIX "A"

SUMMARY OF MASONRY WALLS (SAFETY RELATED)

SERIAL	WALL		WALL DES		WALL'S PRINGRY FUNCTION	SAFETY RELATED SYSTEMS/EQUIPMENT ATTACHED TO WALL (II)	SAFETY RELATED SYSTEMS/EQUIPMENT IN PROXINITY OF WALL (II)	TOTATIVE RE-EVALUATION
NO	NO (1)	BUILDING	ELEV.	LOCATION	Porchar	100000000000000000000000000000000000000	PRANTI G MAD (117)	PRIORITY
1	15.1	Radwaste	88-0 & 91-6	Standby Gas Treatment Room	Shielding	7	Standby Gas Treatment System (Fans, Filters and Ducts)	3
2	15.2	-DO-	-00-	-DO-	-DO-	7	-10-	3
3	16.1	-DO-	-DO-	Reactor Core Isolation Cooling Pump Area, Unit 2	-DO-	None	Reactor Core Isolation Cooling Pump Turbine, Unit 2	3
	16.2	-DO-	-00-	-DO-, Unit 3	-00-	None	-DO-, Unit 3	3
5	16.3	-DO-	-00-	Waste Sludge Tank Room	-DO-	None .	Safeguard Related Conduit	3
6	32.1	-DO-	116-0	Concensate Tank & Pump Rooms	-DO-	7, 10	Mone	3
7	32.2	-DO-	-DO-	-DO-	-DO-	None	480V AC Power	3
8	32.3	-DO-	-DO-	Condensate Pump Area	-DO-	1, 7	BGT & RCIC	3
9	32.4	-DO-	-DO-	-DO-	-DO-	-DO-	-DO-	3 /
10	32.5	-00-	-DO-	-DO-	-DO-	-DO-	-10-	3
11	32,10	-DO-	-00-	Filter Holding Pump Room & Fuel Pool Filter Demineralizer Area	Chielding .	7	Mone	3
12	32.11	-DO-	-DO-	-DO-	-DO-	None	•	3
13	32.12	-DO-	-DO-	-DO-	- DO-	None	.7.	3
14	56.1	-DO-	135-0	RW Control Room & Filter Aid Tank & Pump Area	-DO-	6	Mone	3
15	78.3	-DO-	165-0	Redwhete Building "	-DO-	17	Emerg. Swgr Filter Mousing	3
16	25.1	Turbine (Con- trol Structure)	116-0	Switchgear & Conventional Chem. Lab	Partition	2, 3, 8 thru'll	High Pressure and Emergency Service Water Lines and Safeguard Conduits	2
17	25.2	-00-	-00-	-DO-	-DO-	2, 3	-00-	2
18	40.1	-DO-	135-0	Battery Room, Unit 2	Fire Resistance	8, 10, 12, 14	'Buttery Rack, Channels A & C, Unit 2	1

APPENDIX A

PEACH BOTTOM ATOMIC FOWER STATION UNITS 2 6 3

SERIAL	WALL	NAUL DESCRIPTION			WALL'S PRIVERY	SAFETY RELATED SYSTEMS/EQUIPMENT	SAFETY RELATED SYSTEMS/BOULDHENT IN	TENTATIVE
NO	NO (I)	BUILDING	ELEV.	LOCATION	FUNCTION	ATTACHED TO WALL (II)	PROXIDITY OF WALL (II)	PRIORITY
19	40.2	Turbine (Control Structure)	135-0	Switchgear Room Unit 2	Fire Resistance	2, 7, 9, 10, 12, 13, 14, 17	h kV Switchgear, Channel C, & MCC Channel C, Unit 2	,
50	40.3	-DO-	-DO-	-DO-	-DO-	1, 2, 3, 6 thru' 14, 17.	4 kV Switchgear, Channel D, Unit 2	1
21	40.4	-00-	-DO-	-50-, Unit 3	-100-	2, 8, 9, 11, 12,13, 17	-DO-, Channel D, Unit 3	1
55	40.5	-DO-	-DO-	-DO-, Unit 3	-DO-	2, 3, 5, 6, 7, 9, 10, 12, 13,	-DO-, Channel C, Unit 3	1
23	40.6	-DO-	-DO-	Battery Room, Unit 3	-DO-	1, 8, 10, 12, 14	Battery Rack, Channels A & C, Unit 3	1
24	40.7	-DO-	-DO-	Battery Room, Unit 2	-DO-	10, 12, 14	Battery Rack, Channels A, B, C, & D, Unit 2 and Fuse Boxes Channels A&C.	1
25	40.8	-DO-	-DO-	Switchgear Room, Unit 2	-DO-	7, 10, 12, 14	4 kV Switchgear, Channels A & C, Unit 2	1
26	40.9	-DO-	-DO-	-DO-, Unit 2	-DO-	7, 9 thru'13	-DO-, Channels B & D, Unit 2	1
27	40.10	-DO-	-DO-	-DO-, Unit 3	-DO-	8, 9, 10, 17	-DO-, Channels B & D, Unit 3	1
28	40.11	-DO-	-DO-	-DO-, Unit 3	-DO-	2, 3, 7 thru'12, 14, 17	-DO-, Channels A & C, Unit 3	1
29	40.12	-DO-	-DO-	Bettery Room, Unit 3	-DO-	1, 8, 10, 12	Battery Rack, Channels A, B, C, & D, Unit 3 and Fuse Boxee Channels A&C	1
30	40.13	-00-	-00-	Battery Room & Switch- gear, Unit 2	-DO-	6, 8 thru 12, 14, 17	Battery Rack, Channels A & C, MCC Channel C, Fuse Boxes Channels, A & C 4 kV Switchgear, Channel C, Unit 2	1
31	40.14	-00-	-DO-	-DO-	-DO-	2, 8, 10, 11, 12, 14	Battery Rack, Channels B & D, MCC Channel A, Fuse Boxes Channels B & D. 4 kV Switchgear, Channel A, Unit 2	1
32	40.15	-DO-	-DO-	Switchgear Room, Unit 2	-DO-	2, 9, 11, 12	4 kV Switchgear, Channels C & D, Unit 2	1
33	40.16	-00-	-DO-	Switchgear Room, Unit 2	-DO-	2, 8 thru'l4	4 kV Switchgear, Channels A & B, Channel B, Unit 2	1
34	40.17	-DO-	-DO-	Switchgear Room, Units 2 & 3	-10-	2, 8 thru'l3	4 kV Switchgear, Channels D & D, Units 2 & 3	1 . 1

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ERIAL NO	NO (I)	-	- WALL D	ESCRIPTION	WALL'S PRIMERY SAFETY RELATED SYSTEMS POULTMENT			
	mo (1)	BUILDING	- ELEV	LOCATION	FUNCTION FUNCTION	SAFETY RELATED SYSTEMS/EQUIPMENT ATTACHED TO WALL (II)	SAFETY RELATED SYSTEMS/BUILDMENT IN PROXIMITY OF WALL (II)	TENTATIVE PROPERTY
35	40.18	Turbine (Control Structure)	135-0	Switchgear Room, Units 2	Fire Resistance	2, 9, 11, 12	4 kV Switchgear, Channels B & B, Units 2	PRIORITY
6	40.19	-DO-	-00-	Switchgear Room, Unit 3	-Do-		43, Units 2	1
	40.20	-00-	-DO-	-DO-	-00-	8, 9, 11, 12, 17	-DO-, Channels C & D, MCC Channel D,	1
1	40.21	-DO-	-DO-	Switchgear & Battery	-DO-	2, 7, 8, 9, 11 thru' 14	-DO-, Channels A & B, MCC Channel B,	1
1	40.22	-00-	-DO-	Rooms, Unit 3	-DO-		Battery Rack, Channels A & C, Fuse Boxes Channels A & C 4 kV Switchgear, Channel C, Unit 3	1
	40.23	-DO-	-DO-	Switchgear Room, Unit 2	-Do-	4, 8, 10, 11, 12, 14	Battery Rack, Channels B& D, MCC Channel A, Buse Boxes, Channels B & D kV Switchgear, Channel A, Unit 3	1
1	40.24	-00-	-DO-	-DO-, Unit 2			4 kV Switchgear, Channel D, Unit 2 and	3
1	40.25	-DO-	-DO-	-DO-, Unit 3	-D0-	None	-00-	
1	40.26	-DO-	-DO-	-DO-, Unit 3	-DO-	None	-DO-, Unit 3	3
1	40.27	-DO-	-DO-	-DO-, Unit 2	-DO-	None	-00-	,
'	40.28	-DO-	-DO-	-DO-	-DO-	None	kV Switchgear & MCC Channel A, Unit 2	,
1	10.29	-DO-	-DO-	-DO-, Unit 3	-00-	Hone 4	kV Switchgear & MCC Channel B, Unit 2	3
1	10.30	-DO-	-DO-	-DO-	-00-	None 4	kV Switchgear & MCC Channel B, Unit 3	3
	8.1	-DO-	150-0	Cable Spreading Room	-DO-		kV Switchgear & MCC Channel A, Unit 3	3
	8.2	-DO-	-DO-	-DO-	-DO-	1, 8, 9, 11, 13, 17	afeguard Trays	1
	8.3	-DO-	-DO-	-DO-	-DO-	1 thru's 7 thru's at an	-10-	2 11
	3.4	-DO-	-DO-	-DO-		1 thm/7 0 m 100	-DO- gr	1
680	3.5	-DO-	-DO-	-D0-		None	-20-	1 2
								2

APPENDIX "A"

SUMMARY OF MASONRY WALLS (SAFETY RELATED)

SERIAL	WALL		WALL DES	SCRIPTION	WALL'S PRIMIRY	SAFETY RELATED SYSTEMS/EQUIPMENT	I make the second of the secon	TENTATIVE
NO	NO (I)	BUILDING	ELEV.		FUNCTION	ATTACHED TO WALL (II)	PROXIMITY OF WALL (II)	PRIORITY
53	71.1	Turbine (Control Structure)	165-0	Control Room	Shielding	None	Control Panels, Safeguard Trays	2
54	45.1	. Reactor, Unit 2	135-0	Isolation Valve Comp.	-DO-	2, 3, 5	a) Containment Isolation Valves and Assoc. Conduits b) CRD Insert and Withdrawal Lines	1
55	15.2	-DO-	-DO-	-10-	-DO-	2, 3, 4, 10, 15	-DO-	1
56	45.4	-00-	-DO-	Steam Pipe Tunnel	-DO-	5, 15, 18	MSIV on Steam Line "A" and Related Conduits	2
57						None	5, 15, 18	2
57	45.6 75.6	-DO-	165.0	-DO-	-DO-			
50	75.0	-00-	165-0	Isolation Valve '.es	-DO-	Hone	& RNCU leakage detection temp. element	
59	76.6	-00-	-DO-	Load Centers	Fire Resistance	9, 10	Electrical Load Center	1.4
60	76.7	-00-	-DO-	-10-	*D0-	None	-DO-, 1 thru 7, 9, 10, 15, 17	1
61	76.8	-DO-	-DO-	-10-	-DO-	None	Electrical Load Center	4
62	76.9	-DO-	-DO-	-DO-	-DO-	None .	-DO-, 1 thru 7, 9, 10, 15	1 '
63	76.10	-DO-	-DO-	-DO-	-DO-	2 ,	Electrical Load Center	
64	87.1	-DO-	180-0	Isolatica Valva Compartment	Shielding	None	Containment Isolation Valves, RWCU Leakage Detection Tamp. Element	
65	102.8	-10-	234-0	Refueling	-10-	6	None	
66	102.9	-DO-	-DO-	-bo-	-10-	6	None	2
67	406.1	Reactor, Unit 3	135-0	Isolation Valve Comp.	-10-	2, 3, 15	a) Containment Isolation Valves & Conduits b) CRD insert and Withdrawal Lines	1
68	406.2	-DO-	-DO-	-DO-	-DO-	2 thru' 5, 7, 12	-10-	1

APPENDIX A

SUMMARY OF MASONRY MALLS (SAFETY RELATED)

SERIAL	WALL	THE RESIDENCE OF	WALL DES	CRIPTION	WALL'S PRIMING	SAFETY RELATED SYSTEMS/EQUIPMENT		TENTATIVE
NO	NO (I)	BUILDING	ELEV.	LOCATION	FUNCTION 1 1 1 1	ATTACHED TO WALL (II)	PROXIMITY OF WALL (II)	PRIORITY
-								
69	406.6	Reactor, Unit 3	135-0	Steam Pipe Tunnel	Shielding	None	MSIV on Steam Line "A" and Related Conduits	
70	406.9	-DO-	-DO-	-DO-	-DO-	5, 12	MSIV on Steam Line "A" and Related Conduits	5
71	406.10	-DO-	-DO-	-DO-	-DO-	3, 5, 10, 12	-100-	1
72	409.7	-D0-	165-0	Isolation Valve Area	-DO-	None	Containment Isolation Valve & Associated Conduits and RWCU Leakage Detection Temperature Element	
73	410.1	-00-	-DO-	Regenerative Heat Ex- changer Area	-00-	,	None	4
74	410.6	-DO-	-DO-	Load Center	Fire Resistance	None	Electrical Load Centers, 7,10,11	2
75	410.7	-DO-	-DO-	-DO-	-DO-	None	-10-, 2,3,5,6,7,10,11,14,15	1
76	410.8	-DO-	-DO-	-DO-	-DO-	None	-00-, 10,12	2
77	410.9	-00-	-DO-	-DO-	-DO-	None	-DO-, 2,3,5,7,10,11,12,14,17	1
78	410.10	-DO-	-DO-	-DO-	-DO-	None	-DO-, 10,11,17	2
79	412.1	-00-	180-0	Isolation Valve Compartment	Shielding	None	Containment Isolation Valve and Conduits & RWCU Leakage Detection Temp. Element	4
80	418.10	-DO-	234-0	Refueling Floor	-00-	6	None	4
81	418.11	-00-	-DO-	-DO-	-DO-	6	-DO-	4
82	532.1	Emergency Cooling Tower	158-0	Emergency Cooling Tower	Fire Resistance	4,8	Emergency Cooling Water Pump and Load Centers and Panels, MCC	•

Sheet 6 of 7

SUMMARY OF MASONR' WALLS (SAFETY RELATED)
PEACH BOTTOM ATOMIC FOWER STATION INITS 2 & 3

TEMTATIVE	PRIORITY	•	•	•	•	
SAFETY ALLAND SYSTEM (BUTTHENT IN	PRODUCTY OF WALL (11)	Load Centers and Panele, MCC	ą	Emergency Service Water Piping	· ·	
SAFETY RELATED SYSTEMS/EQUITMENT	ATTACHED TO WALL (II)	60		None	None	
WIT'S PRIMIN	FUNCTION	Fire Restatance	-04-	-00-	-D0-	
WAL DESCRIPTION	LOCATION	Emergency Cooling Tower	4	Pump Room	-Bo-	
WAL DES	ELEV.	158-0	8	112-0	-00-	
	BUILDING	Energency Cooling Tower	-82-	Circulating Water Pump Structure	-04-	
MALL	NO (T)	532.2	532.3	c86.1 C	c86.2	
SERIAL	ON.	83	18	85	98	

APPENDIX "A"

SUMMARY OF MASONRY WALLS (SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 243

MOTES:

I. Each wall is designated by a unique number such that the first part indicates the Civil/Structural Drawing number and the second part gives the wall number on that particular drawing, e.g. wall No. 45.2 indicates wall number 2 on Civil/Structural Drawing No. 45.

II. Safety related systems are designated as below:

- 1. Reactor Core Isolation Cooling (RCIC)
- 2. Reactor Heat Removal (RHR)
- 3. Core Spray
- 4. High Pressure Coolant Injection (HPCI)
- 5. Primary Containment Isolation
- 6. Process/Area Radiation Monitoring
- 7. Standby Gas Treatment
- 8. Emergency for the Water (ESW)
- 9. Emergency b. .el Generator
- 10. Safeguard 4t . VAC Power (MCC, LC)
- 11. Safeguard 4 kV Power
- 12. Safeguard 125/250 VOC Power
- 13. Safeguard 120 VAC Power
- 14. Safeguard 24 VDC Power
- 15. Containment Atmospheric Dilution (CAD)
- 16. Reactor Protection System (RPS)
- 17. Safeguard HVAC System
- 18. Steam Leak Detection System

APPENDIX "B"

ERIAL				ESCRIPTION			
NO.	WALL NO.	BUILDING	ELEVATION	LOCATION	BASIS FOR CLASSIFICATION		
1	24.1	Turbine	116-0	Valve Operating Area	Plant Walkdown & Eng. Review		
2	32.6	Radwaste	-DO-	Condensate Pump & Tank Rooms	Engineering Review		
3	32.7	-DO-	-DO-	-DO-	-DO-		
4	32.8	-DO-	-DO-	-DO-	-DO-		
5	32.9	-DO-	-DO-	-DO-	-DO-		
6	61.1	-DO-	150-0	Hopper Compartment	-DO-		
7	61.2	-DO-	-DO-	-DO-	-DO-		
8	61.3	-DO-	-DO-	Radwaste DH&V Equipment Compartment	-DO-		
9	61.4	-DO-	-DO-	-DO-	DO-		
10	61.5	-DO-	-DO-	-DO-	-DO-		
11	61.6	-DO-	-DO-	-DO-	-DO-		
12	61.7	-DO-	-DO-	-DO-	-DO-		
13	61.8	-DO-	-DO-	-DO-	-DO-		
14	78.1	-DO-	165-0	Centrifuge Room	-DO-		
	i i						

APPENDIX "B"

SERIAL		I HERRICAL TANDA	WALL D	ESCRIPTION	
NO.	WALL NO.	BUILDING	ELEVATION	LOCATION	BASIS FOR CLASSIFICATION
15	78.2	Radwaste	165-0	Centrifuge Room	Engineering Review
16	45.3	Reactor, Unit 2	135-0	Steam Pipe Tunnel	Plant Walkdown & Eng. Rev.
17	45.5	-DO-	-DO-	-DO-	-DO-
18	75.1	-DO-	165-0	Clean Up Recirc. Pump Room	DO-
19	75.2	-DO-	-DO-	-DO-	-DO-
20	75.3	-DO-	-DO-	-DO-	-DO-
21	75.4	-DO-	-DO-	-DO-	-DO-
22	75.5	-DO-	-DO-	-DO-	-DO-
23	75.7	-DO-	-DO-	-DO-	-DO-
24	76.1	-DO-	-DO-	Regen. Heat Exchanger	-DO-
25	76.2	-DO-	-DO-	Non Regen. Heat Exch. Room	-DO-
26	76.3	-DO-	-DO-	-DO-	-DO-
27	76.4	-DO-	-DO-	Transfer Pump Room	-DO-
28	76.5	-DO-	-DO-	Backwash Rec. Tank Room	Plant Walkdown & Eng. Revie

APPENDIX "B"

SUMMARY OF MASONRY WALL (NON-SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIA	L			SCRIPTION	PACTE BOD CLASSIFICATION
NO.	WALL NO. I	BUILDING	ELEVATION	LOCATION	BASIS FOR CLASSIFICATION
29	76.11	Reactor, Unit 2	165-0	Transfer Pump Room	Engineering Review
30	97.1	-DO-	195-0	Prefilter & Filter Compartments	Plant Walkdown & Eng. Review
31	97.2	-DO-	-DO-	-DO-	Plant Walkdown
32	97.3	-DO-	-DO-	-DO-	-DO-
33	97.4	-DO-	-DO-	-DO-	-DO
34	101.1	-DO-	234-0	Refueling Floor	-DO-
35	102.1	-DO-	-DO-	-DO-	-DO-
36	102.2	-DO-	-DO-	-DO-	-DO-
37	102.3	-DO	-DO-	-DO-	-DO-
38	1102.4	-DO	-DO-	-DO-	-DO-
39	102.5	-DO-	-DO-	-DO-	-DO-
40	102.6	-DO-	-DO-	-DO-	-DO-
41	1102.7	-DO-	-DO-	-DO-	-DO-
**	1		1	bb lower a sect. rate bedee	-DO-
42	1102.10	DO-	-00-	DO	-10-

APPENDIX "B"

SUMMARY OF MASONRY WALL (NON-SAFETY RELATED)

PEACH BOTTOM ATOMIC POWER STATION UNITS 2 & 3

SERIAL		MATERIAL SECTION AND ADDRESS OF THE PARTY OF	WALL DI	ESCRIPTION	
NO.	WALL NO.	BUILDING	ELEVATION	LOCATION	BASIS FOR CLASSIFICATION
43	102.11	Reactor, Unit 2	234-0	Refueling Floor	Plant Walkdown
44	406.3	Reactor, Unit 3	135-0	Neutron Monitoring Room	Plant Walkdown & Eng. Rev.
45	406.4	-DO-	-DO-	-DO-	-DO-
46	406.5	-DO-	-DO-	Steam Pipe Tunnel	-DO-
47	406.7	-DO-	-DO-	-DO-	DO-
48	1406.8	-DO-	-DO-	-DO-	-DO-
49	409.1	-DO-	165-0	Clean Up Recirc. Pump Room	-DO-
50	1409.2	-DO-	-DO-	-DO-	-DO-
51	409.3	-00-	-DO-	-DO-	-DO-
52	409.4	-00-	-DO-	-DO-	-DO-
53	1409.5	-DO-	-DO-	-DO-	-DO-
54	409.6	-DO-	-DO-	-DO-	-DO-
55	410.2	-DO-	-DO-	Backwash Rec.Tank Room	Engineering Review
56	1410.3	-DO-	-DO-	Transfer Pump Room	-DO-

APPENDIX "B"

SERIAL			WALL D	ESCRIPTION	A CAMPAGE AND A STATE OF THE ST
NO.	WALL NO.	BUILDING	ELEVATION	LOCATION	BASIS FOR CLASSIFICATION
57	410.4	Reactor, Unit 3	165-0	Non Regen. Heat Exch. Room	Plant Walkdown & Eng. Rev.
58	410 .5	-DO-	-DO-	-DO-	-DO-
59	410.11	-DO-	-DO-	Transfer Pump Room	-DO-
60	413.1	-DO-	195-0	Storage Area	Plant Walkdown
61	413.2	-DO-	-DO-	-DO-	-DO-
62	413.3	-DO-	-DO-	-DO-	-DO-
63	413.4	-DO-	-DO-	-DO-	-DO-
64	413.5	-DO-	-DO-	-DO-	-DO-
65	413.6	-DO-	-DO-	-DO-	-DO-
6 6	414.1	-DO-	-DO-	Prefilter & Filter Compartment	Plant Walkdown & Engr. Revi
67	414.2	-DO-	-DO-	-DO-	Plant Walkdown
	414.3	-DO-	-DO-	-DO-	Ent. 21DO-
-	414.4	-DO-	-DO-	-DO-	-DO-
.0	417.1	-DO-	234-0	Refueling Area	-DO-

APPENDIX "B"

SERIA	I.		WALL DES	CRIPTION	
NO.	WALL NO.	BUILDING	TELEVATION	LOCATION	BASIS FOR CLASSIFICATION
71	418.1	Reactor, Unit 3	234-0	Refueling Area	Plant Walkdown
72	418.2	-DO-	-DO-	-DO-	-DO-
73	418.3	-DO-	-DO-	-DO-	-DO-
74	418.4	-DO-	-DO-	-DO-	-DO-
75	1418.5	-DO-	-DO-	-DO-	-DO-
76	1418.6	-DO-	-DO-	-DO-	-DO-
77	418.7	-DO-	-DO-	-DO-	-DO-
78	1418.8	-DO-	-DO-	-DO-	-DO-
79	1418.9	-DO-	-DO-	-DO-	-DO-
80	418.12	 -DO-	-DO-	-DO-	-DO-
81	1418.13	 -DO-	-DO-	-DO-	-DO-
					1 (28)

^{*} NOTE: Based on Engineering review, all masonry walls of Recombiner Building and masonry walls of Radwaste, Turbine, and Reactor Building (Units 2 & 3) not listed above, are non-safety related.