

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

THE TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 27 License No. NPF-3

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by The Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees) dated December 6, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I:
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part of the Commission's regulations and all applicable requirements have been satisfied.

 Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-3 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 27, are hereby incorporated in the license. The Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert W. Reid, Chief

Operating Reactors Branch #4

Division of Licensing

Attachment: Changes to the Technical Specifications

Date of Issuance: July 7, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 27

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following pages of the Appendices "A" and "B" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Appendix A Pages

6-2

6-7

6-8

6-9

6-11

6-12

6-13

Appendix B Page

5.1-1

6.1 RESPONSIBILITY

6.1.1 The (Plant Superintendent) shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

OFFSITE

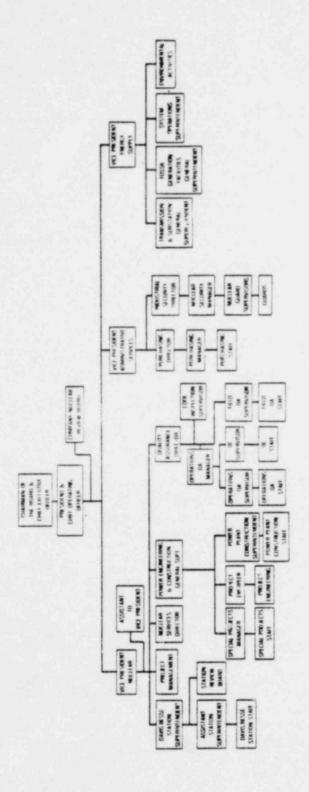
6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

- 6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:
 - a. Each on duty thift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
 - b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
 - c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
 - d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
 - e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
 - **f. A site Fire Brigade of at least 5 members shall be maintained onsite at all times#. The Fire Brigade shall not include 3 members of the minimum Shift crew necessary for safe shutdown of the unit and any personnel required for other essential functions during a fire emergency.

Ffire Brigade Composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of Fire Brigade members provided immediate action is taken to restore the Fire Brigade to within the minimum requirements.

** ... is change from 3 to 5 individuals will be implemented by start up after the first refueling outage.



DAVIS BESSE MUCLEAR POWER STRESSING UNIT NO. 1 OFF SITE ORGANIZATION CHART FIGURE 6.2.1

- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications including preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President Nuclear and to the Chairman of the Company Nuclear Review Board.
- f. Review of events requiring 24 hour written notification to the Commission.
- g. Review of facility operations to detect potential safety hazards
- h. Performance of special reviews, investigations and analyses and reports thereon as requested by the Chairman of the Company Nuclear Review Board.
- i. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Chairman of the Company Nuclear Review Board.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Chairman of the Company Nuclear Review Board.

AUTHORITY

- 6.5.1.7 The Station Review Board shall:
 - a. Recommend to the Station Superintendent written approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
 - b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
 - c. Provide written notification within 24 hours to the Vice President, Nuclear and the Company Nuclear Review Board of disagreement between the SRB and the Station Superintendent; however, the Station Superintendent shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The Station Review Board shall maintain written minutes of each meeting and copies shall be provided to the Vice President, Nuclear and Chairman of the Company Nuclear Review Board.

6.5.2 COMPANY NUCLEAR REVIEW BOARD (CNRB)

FUNCTION

- 6.5.2.1 The Company Nuclear Review Board (CNRB) shall function to provide independent review and audit of designated activities in the areas of:
 - a. Nuclear power plant operations,
 - b. Nuclear engineering,
 - c. Chemistry and radiochemistry,
 - d. Metallurgy,
 - e. Instrumentation and control,
 - f. Radiological safety,
 - g. Mechanical and electrical engineering, and
 - h. Quality assurance practices.

COMPOSITION

6.5.2.2 The Company Nuclear Review Board shall be composed of the:

Chairman: General Superintendent, Power Engineering and

Construction

Member: Vice President, Nuclear

Member: Vice President, Energy Supply

Member: General Superintendent, Transmission and Substations

Member: Superintendent, Davis-Besse Station

Member: Director, Nuclear Services

Member: Nuclear Project Engineer, Power Engineering

Member: Nuclear Engineer, Power Engineering

Member: Director, Quality Assurance

Member: General Superintendent, Fossil Generation Facilities

Member: Others as deemed advisable by the CNRB Chairman*

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the CNRB Chairman to serve on a temporary tasis; however, no more than two alternates shall participate as voting members in CNRB activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the CNRB Chairman to provide expert advice to the CNRB.

MEETING FREQUENCY

6.5.2.5 The CNRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per six months thereafter.

QUORUM

6.5.2.6 A quorum of CNRB shall consist of the Chairman or his designated alternate and at least half of the appointed CNRB members or their alternates. No more than a minority of the quorum shall have line responsibility for operation of the facility.

*Others as dremed advisable by the CNRB chairman, who are appointed to the Company Nuclear Review Board shall have an academic degree in an Engineering or Physical Science Field; and in addition, shall have a minimum of five years of technical experience, of which a minimum of three years shall be in one or more of the areas specified in Specification 6.5.2.1.

REVIEW

- 6.5.2.7 The Company Nuclear Review Board shall review:
 - a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
 - b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
 - c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
 - d. Proposed changes in Technical Specifications or this Operating License.
 - e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
 - f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
 - g. Events requiring 24 hour written notification to the Commission.
 - h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
 - i. Reports and meetings minutes of the Station Review Board.

AUDITS

- 6.5.2.8 Audits of facility activities shall be performed under the cognizance of the CNRB. These audits shall encompars:
 - a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
 - b. The performance, training and qualifications of the entire station staff at least once per 12 months.
 - c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
 - d. The performance of activities required by the Quality
 Assurance Program to meet the criteria of Apperdix 'B", 10 CFR
 50, at least once per 24 months.
 - e. The Station Emergency Plan and implementing procedures at least once per 24 months.
 - f. The Station Security Plan and implementing procedures at least once per 24 months.
 - g. Any other area of facility operation considered appropriate by the CNRB.
 - h. The Facility Fire Protection Program and implementing procedures at least once per 24 months.
 - i. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offs te licensee personnel or an outside fire protection firm.
 - j. An inspection and audit of the fire protection and loss prevention program shall be performed by a qualified outside fire consulant at least once per 36 months.

AUTHORITY

6.5.2.9 The Company Nuclear Review Board shall report to and advise the President and Chief Operating Officer on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

RECORDS

- 6.5.2.10 Records of Company Nuclear Review Board activities shall be prepared, approved and distributed as indicated below:
 - a. Minutes of each CNRB meeting shall be prepared, approved and forwarded to the President and Chief Operating Officer and CNRB members within 14 days following each meeting.
 - b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the President and Chief Operating Officer and CNRB members within 14 days following completion of the review.
 - c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the President and Chief Operating Officer and CNRB members and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

- 6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:
 - a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
 - b. Each REPORTABLE OCCURRENCE requiring 24 hour notification to the Commission shall be reviewed by the SRB and submitted to the CNRB.

6.7 SAFETY LIMIT VIOLATION

- 6.7.1 The following actions shall be taken in the event a Safety Limit is violated:
 - a. The facility shall be placed in at least HOT STANDBY within one hour.
 - b. The Safety Limit violation shall be reported to the Commission, the Vice President, Nuclear and to the CNRB within 24 hours.
 - c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SRB. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
 - d. The Safety Limit Violation Report shall be submitted to the Commission, the CNRB and the Vice President, Nuclear within 14 days of the violation.

6.8 PROCEDURES

- 6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:
 - a. The applicable procedures recommended in Appendix "A Regulatory Guide 1.33, November, 1972.
 - b. Refueling operations.
 - c. Surveillance and test activities of safety related equipment.
 - d. Security Plan implementation.
 - e. Emergency Plan implementation.
 - f. Fire Protection Program implementation.
- 6.8.2 Each procedure of 6.8.1 above, and changes thereto, shall be reviewed by the SRB and approved by the Station Superintendent prior to implementation and reviewed periodically as set forth in administrative procedures.

- 6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:
 - a. The intent of the original procedure is not altered.
 - b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reacter Operator's License on the unit affected.
 - c. The change is documented, reviewed by the SRB and approved by the Station Superintendent within 14 days of implementation.

6.9 REPORTING REQUIREMENTS

ROUTINE REPORTS AND REPORTABLE OCCURRENCES

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Director of the Regional Office of Inspection and Enforcement unless otherwise noted.

STARTUP REPORT

- 6.9.1.1 A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.
- 6.9.7.2 The report shall address each of the tests identified in the FSAR and shall include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.
- 6.9.1.3 Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power (eration, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial

5.0 ADMINISTRATIVE CONTROLS

5.1 Review and Audit

Review and audit of environmental matters and compliance with these Environmental Technical Specifications in particular shall be provided by the Station Review Board, the Company Nuclear Review Board, and the Quality Assurance Director. This activity shall be controlled in accordance with QAP 5000 of the Toledo Edison Nuclear Quality Assurance Manual.

5.1.1 Station Review Board

A Station Review Board, composed of technically qualified station personnel has been established to perform timely and continuing reviews of unit Operation. The makeup of the Board, specification of quorum, and meeting frequency are set forth in Section 6.0 of the Unit's Appendix "A" Technical Specifications. The Board's review responsibility will in lude all new or revised unit environmental procedures, proposed changes or modifications to unit structures or equipment, reported violations of the unit's Environmental Technical Specifications, proposed changes to the unit's Environmental Technical Specifications, results of the Environmental Monitoring Programs prior to their submission in each Environmental Operating Report, and any occurrence of a safety limit being exceeded.

5.1.2 Company Nuclear Review Board

A Company Nuclear Review Board, composed of technically qualified personnel, has been appointed to perform independent reviews of unit operation. The makeup of the Board, specification of quorum and meeting frequency are set forth in Section 6.0 of the Unit's Appendix "A" Technical Specifications. The Board's independent review responsibility will include all new or revised unit environmental procedures, proposed changes or modifications to unit structures or equipment, reported violations of the unit's Environmental Technical Specifications, proposed changes to the unit's Environmental Technical Specifications, station operation, and minutes of the Station Review Board meeting.

5.1.3 Quality Assurance Director

The Quality Assurance Director shall be responsible for verification of compliance with the Environmental Technical Specifications. Planned periodic reviews and audits will be conducted in accordance with the provisions of the quality assurance program for unit operation described in the unit's Final Safety Analysis Report. The Quality Assurance Director shall also be responsible for reviewing all nonconformance reports concerning Environmental Technical Specifications including corrective actions taken to prevent any recurrence of the same nonconformance.